

## Fission Gas Release from Accident Tolerant Fuels: A Review

Yang-Hyun Koo\*, Jang-Soo Oh, Hyo-Chan Kim, Dong-Joo Kim, Jae-Ho Yang, Hyun-Gil Kim

Korea Atomic Energy Research Institute, 111, Daedeok-daero 989 Beon-gil, Yuseong-gu, Daejeon, 34057, Korea

\*Corresponding author: yhkoo@kaeri.re.kr

\***Keywords:** Accident tolerant fuel, microcell pellet, Cr<sub>2</sub>O<sub>3</sub>-doped pellet, fission gas release

### Abstract

After the Fukushima accident that occurred in March 2011 [1], several countries including Korea, US, and France have developed accident tolerant fuel (ATF) that could mitigate the consequences of nuclear accidents [2]. In the case of accident tolerant pellet, it is supposed to maintain reliability during normal operation and moreover exhibit enhanced retention of fission products even in severe accidents, since such pellets could minimize the release of radioactive materials into the environment.

It is noted that fuel pellet is both the source of radioactive fission products and the first barrier for blocking their release to the environment. KAERI has developed microcell UO<sub>2</sub> pellets to enhance the retention capability of highly radioactive and corrosive fission products [3], especially volatile cesium (Cs) and iodine (I), which are very hazardous in terms of public health. The key concept of the microcell UO<sub>2</sub> pellets is both to increase thermal conductivity for decreasing fuel temperature and also immobilize Cs and I by providing multiple chemical traps inside the pellet [4]. In order to confirm whether the microcell pellets would perform its intended function of increased retaining capabilities, two kinds (metallic and ceramic) of microcell pellets together with the conventional UO<sub>2</sub> pellets were irradiated in the Halden reactor until achieving an average pellet burnup of 16 MWd/kgU [5] at the time of the Halden reactor's permanent shutdown.

Another concept of ATF pellet developed by AREVA and Westinghouse involves modifying the conventional UO<sub>2</sub> pellet with small additives of Cr<sub>2</sub>O<sub>3</sub>, with increasing grain size up to 7 times great than the undoped UO<sub>2</sub> pellet for suppressing fission gas release (FGR) at extended burnup and power ramps, enhancing the accident tolerance [6]. Different kinds of Cr<sub>2</sub>O<sub>3</sub>-doped pellets were irradiated in the Halden reactor (IFA-677 and IFA-716) up to an average burnup of around 30 MWd/kgU, during which fuel temperature and rod pressure were measured online [6,7]. And fission gas releases of these pellets were inferred from the measured rod pressure.

We review the concepts of accident tolerant fuel pellets

developed so far and their irradiation test results, and to evaluate how much the accident tolerant pellets could reduce FGR, compare their FGRs during irradiation with those of the UO<sub>2</sub> pellet [6-8]. Finally, we discuss how the accident tolerant pellets would impact on accident source term during accidents.

### ACKNOWLEDGEMENTS

This work was supported by the Ministry of Science and ICT, Republic of Korea (RS-2022-00144002).

### REFERENCES

- [1] Yang-Hyun Koo, Yong-Sik Yang, Kun-Woo Song, Radioactivity release from the Fukushima accident and its consequences: A review, Prog. Nucl. Energy 74 (2014) 61.
- [2] Shannon Bragg-Sitton, Metrics development for enhanced accident tolerant LWR fuels, OECD/NEA Meeting on Increased Accident Tolerance of Fuels for LWRs, Issy-les-Moulineaux, France, October 28-29, 2013.
- [3] Dong-Joo Kim, Young-Woo Rhee, Jong-Hun Kim, Keon-Sik Kim, Jang-Soo Oh, Jae-Ho Yang, Yang-Hyun Koo, Kun-Woo Song, Fabrication of microcell UO<sub>2</sub>-Mo pellet with enhanced thermal conductivity, J. Nucl. Mater. 462 (2015) 289.
- [4] Dong-Joo Kim, Keon-Sik Kim, Dong-Seok Kim, Jang-Soo Oh, Jong-Hun Kim, Jae-Ho Yang, Yang-Hyun Koo, Development status of microcell UO<sub>2</sub> pellet for accident-tolerant fuel, Nucl. Eng. Tech. 50 (2018) 253.
- [5] K.I. Bjork, J. Kelly, C. Vitanza, S. Drera, S. Holcombe, T. Tverberg, H. Tuomisto, J. Wright, M. Sarsfield, T. Blench, J.H. Yang, H.G. Kim, D.J. Kim, Irradiation testing of enhanced uranium oxide fuels, Ann. Nucl. Energy 125 (2019) 99.
- [6] Yifeng Che, Giovanni Pastore, Jason Hales, Koroush Shirvan, Modeling of Cr<sub>2</sub>O<sub>3</sub>-doped UO<sub>2</sub> as a near-term accident tolerant fuel for LWRs using the BISON code, Nucl. Eng. Des. 337 (2018) 271.
- [7] Michael Cooper, Giovanni Pastore, Yifeng Che, Fission gas diffusion and release for Cr<sub>2</sub>O<sub>3</sub>-doped UO<sub>2</sub>: From the atomic to the engineering scale, J. Nucl. Mater. 545 (2021) 152590.
- [8] Yang-Hyun Koo, Chang-Hwan Shin, Sang-Chae Jeon, Dong-Seok Kim, Keon-Sik Kim, Dong-Joo Kim, Kun-Woo Song, Dong-Hak Kook, Hyun-Gil Kim, Yang-Il Jung, Jae-Ho Yang, Fission gas release in the microcell fuel pellet under normal operating conditions: A simplified approach based on UO<sub>2</sub> pellet experience, J. Nucl. Mater. 527 (2019) 151801.