

# Prediction of the critical flow rate of subcooled water through the short length channel using analytical method

2021. 10. 22

**Taewoo Kim<sup>a</sup>, Hyun-Sik Park<sup>a,b</sup>, Sang Ji Kim<sup>c</sup>**

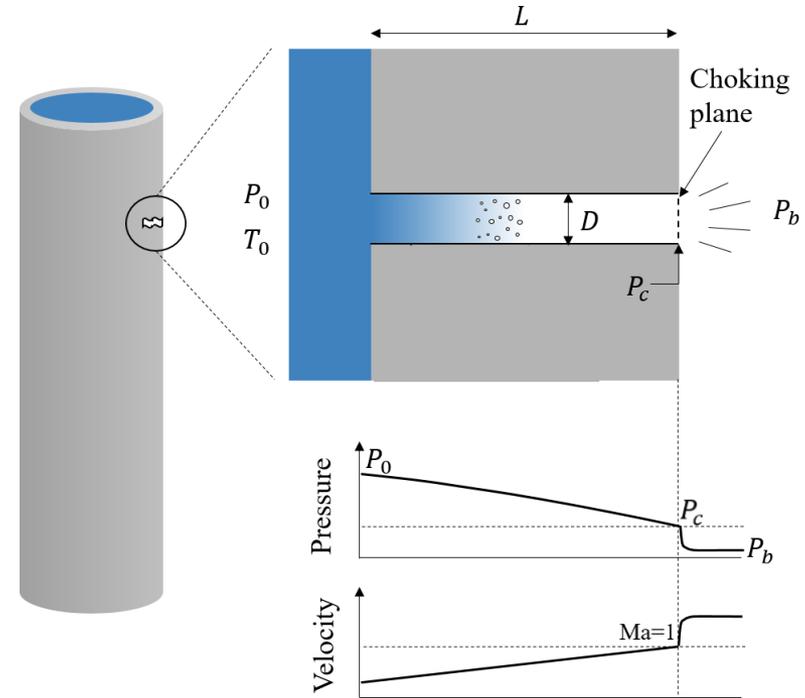
<sup>a</sup> Advanced Nuclear Engineering and System, University of Science and Technology

<sup>b</sup> Innovative System Safety Research Division, Korea Atomic Energy Research Institute

<sup>c</sup> Versatile Reactor Technology Development Division, Korea Atomic Energy Research Institute

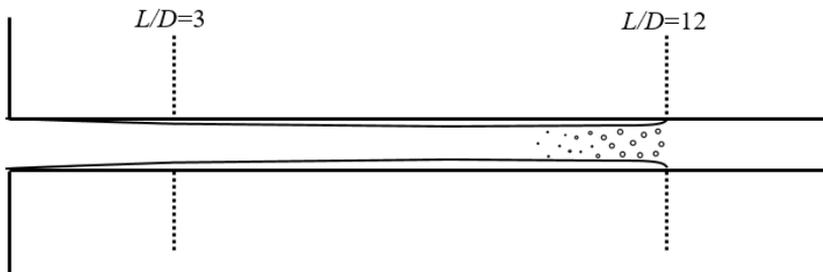
## Two-phase critical flow

- The leakage of pressurized water through breaks is encountered in pipe line and heat transfer tube in nuclear power plants.
- When the velocity of leaking fluid reaches the sound speed, the leak rate cannot be increased more even though the downstream pressure decreases more (critical flow).
- The critical flow rate depends on the stagnation condition and channel geometry. Moreover, the two phase flow occurs due to the flashing of water.
- Most of studies of critical flow are related to long channel ( $L/D > 12$ ), which are not suitable for the SG tubes ( $L/D$  ratios are generally 0.8~2.0) [1].



[ Phenomenon of subcooled water critical flow through the penetrating crack on SG tube ]

## Flow patterns of critical flow [2]



[ Flow patterns of initially subcooled liquid flowing through a sharp inlet ]

$L/D < 3$  : Superheated liquid jet is surrounded by a vapor annulus

$3 < L/D < 12$  : Vapor bubbles are formed in the middle of the jet

$12 < L/D$  : Dispersed two-phase region

## Parameter range of two-phase critical flow experiments on short length channel

	Geometry	Channel length (mm)	Hydraulic diameter (mm)	L/D	P <sub>0</sub> (MPa)	ΔT <sub>sub</sub> (°C)	Remarks
Sozzi and Sutherland (1975) [3]	Tube	4.7-1778	12.7	0.37-140	~6.5	2-43	Henry-Fauske model was not comparable for ΔT <sub>sub</sub> > 20°C
Park et al. (2000) [4]	Tube	2.0-8.0	2.0-8.0	0.5-2.0	4	2-200	Empirical correlation was derived
Revankar et al. (2013) [1]	Slit	1.3	0.59-1.04	1.25-5.42	~6.8	14-51	RELAP5 H-F and R-T model: 30% and 15% error Burnell correlation was modified
Revankar et al. (2019) [5]	Crack	1.2-3.18	0.21-0.65	1.85-6.09	1.4~6.8	12-62	-

- Correlation of two-phase critical flow rate (L/D < 3)

Park et al. [4] 
$$G_c = C_{d,ref} \sqrt{2\rho_{ref}(P_0 - P_b)} \left\{ 1.04 - \frac{3.3}{1 + \exp[(\Delta T_{sub}^* + 1.1)/0.49]} \right\}, \quad \Delta T_{sub}^* = \frac{T_{sat} - T_0}{T_{sat} - T_{ref}}$$

Revanakr et al. [1] 
$$G_c = \sqrt{2\rho_0(P_0 - kP_b)}, \quad k = 1 + 11.6 \left( \frac{\Delta T_{sub}}{T_{sat}} \right)^{1.7}$$

## Limitations of existing models and correlations

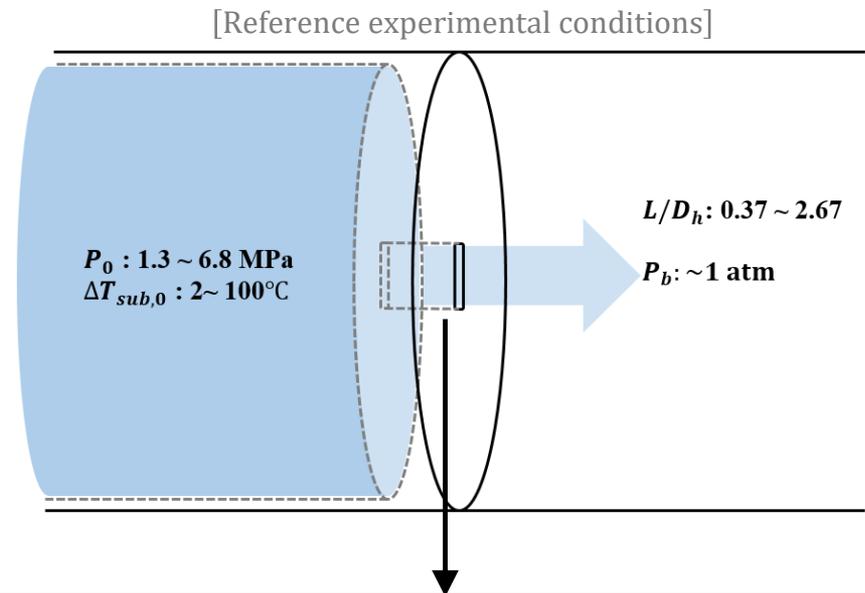
- Analytical models such as Henry-Fauske and Trapp-Ranson model cannot be used for the critical flow through short length channel. Most of models were developed for the range of L/D > 12.
- Although the correlations were developed for L/D > 12, they are not evaluated for various geometries. The developed correlation might be applied only for the specific geometry.

## Objectives

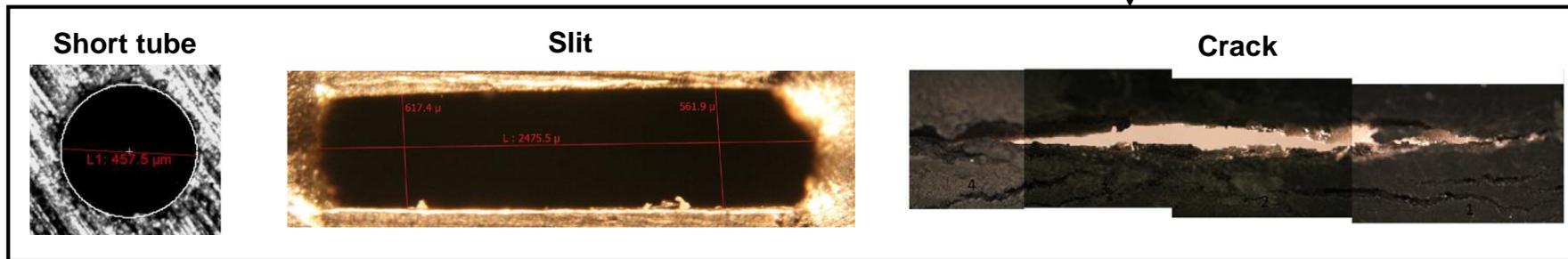
- To investigate the characteristics of critical flow through short length channels
- To develop the analytical method to predict two-phase critical mass flux

## Scope

- Development of analytical method for critical mass flux
  - Problems of Henry-Fauske Model
  - Modification of existing analytical method
- Comparison with the previous
  - Experiments that measured the critical mass flux through short tube, slit, and crack
  - Correlations developed for  $L/D < 3$



### Type of flow channel



→ Ideal momentum equation

$$G = \sqrt{\frac{2(P_0 - P_c)}{v_{f,0}}} \quad \dots (eq. 1)$$

→ Henry-Fauske model for subcooled water [6]

$$G = \left[ (v_{g,eq} - v_{f,0}) \frac{N}{(s_{g,eq} - s_{f,eq})} \frac{ds_{f,eq}}{dP_t} \right]^{-1/2} \quad \dots (eq. 2)$$

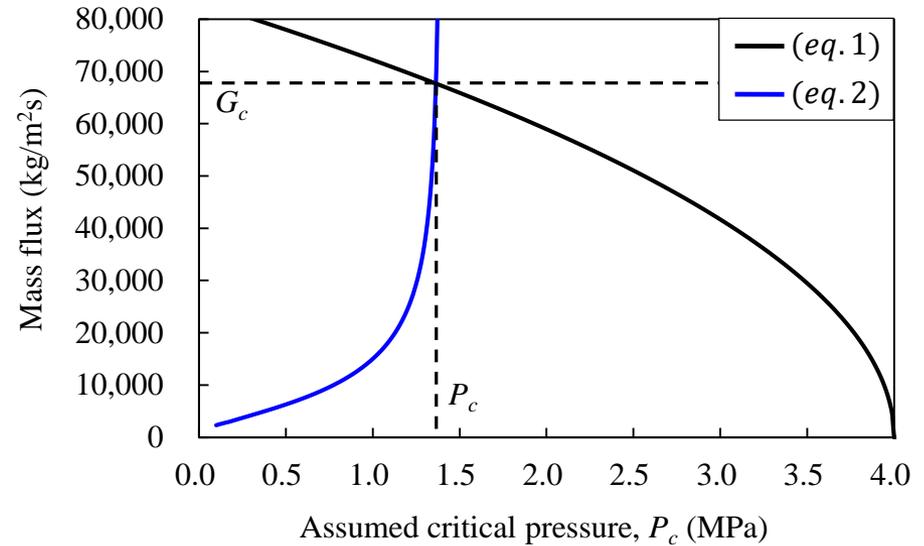
$$N = \begin{cases} x_{eq}/0.14 & x_{eq} \leq 0.14 \\ 1 & x_{eq} > 0.14 \end{cases}$$

- By assuming  $P_c$ , the critical mass flux and pressure are determined at the point where the mass flux calculated by eq. (1) and eq. (2) become equal.

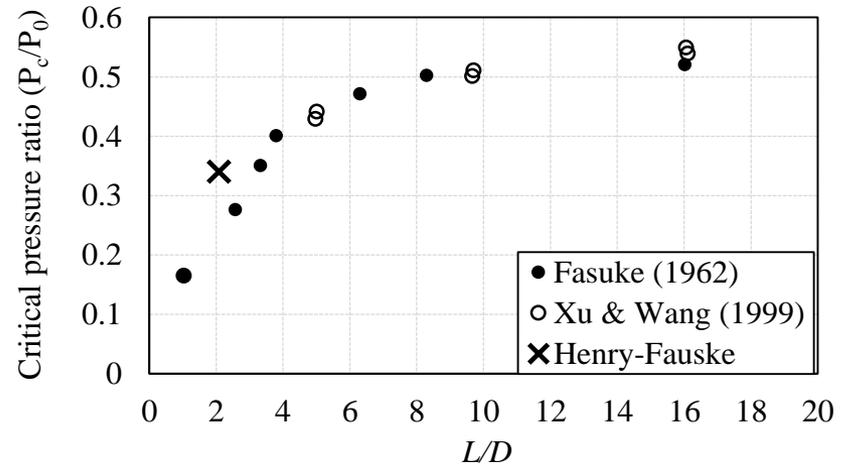
- Henry-Fauske model = 67,589 kg/m<sup>2</sup>s  
Experimental result = 54,400 kg/m<sup>2</sup>s

- Critical pressure ratio is higher than previous investigations

Henry-Fauske model	Fauske's experimental result
$P_c/P_0=0.34$	$P_c/P_0=0.28$



Henry-Fauske model result, Calculation condition [4]  
:  $P_0=4.0$  MPa,  $\Delta T_{sub,0}=55.1$  K,  $L=8$  mm,  $D=4$  mm,  $L/D=2.0$



[Critical pressure ratio of initially saturated water flow through the nozzle [7]]

→ Henry-Fauske model for subcooled water [6]

$$G = \left[ (v_{g,eq} - v_{l,0}) \frac{N}{(s_{g,eq} - s_{l,eq})} \frac{ds_{l,eq}}{dP} \right]^{-0.5}$$

- Wang et al. [8] suggested the following non-equilibrium factor

$$N = 0.0376 \frac{L}{D} - 0.163 \quad (\text{Saturated})$$

$$N = \left( 0.0376 \frac{L}{D} - 0.163 \right) \exp(-0.322 \Delta T_{sub}) \quad (\text{Subcooled})$$

They were compared with experiments ( $L/D = 16.1, 25.6$ )  
**but, cannot be used for  $L/D < 4.34$**

- Ghosh et al. [9] suggested that the critical pressure should be calculated by considering various pressure losses.

$$\Delta P_{tot} = P_0 - P_c$$

$$\Delta P_{tot} = \Delta P_e + \Delta P_f + \Delta P_{aph} + \Delta P_{aa} + \Delta P_K$$

$$\Delta P_e = \text{Entry loss}$$

$$\Delta P_f = \text{Friction pressure drop}$$

$$\Delta P_{aph} = \text{Acceleration pressure drop after flashing}$$

$$\Delta P_{aa} = \text{Pressure loss by area change}$$

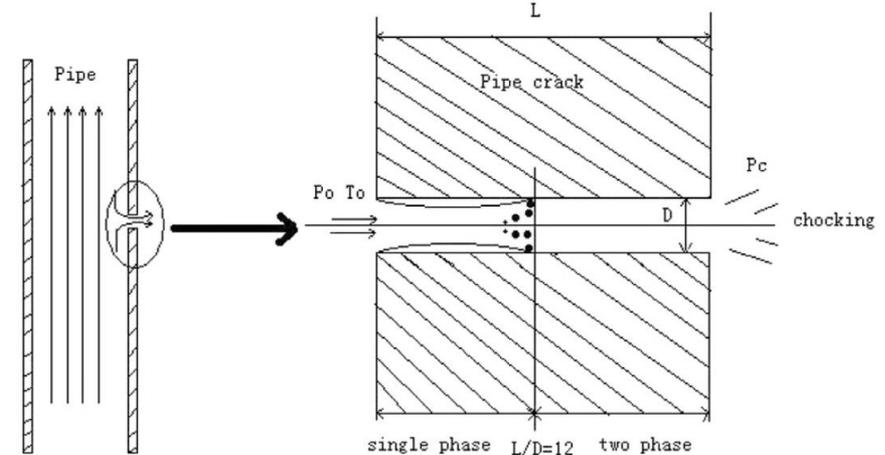
$$\Delta P_K = \text{Pressure loss by protrusion in actual crack}$$

**Some loss terms are not significant for  $L/D < 3$**

**Some loss terms are difficult to be calculated**

→ Ideal momentum equation

$$G = \sqrt{\frac{2(P_0 - P_c)}{v_{f,0}}}$$



[ Diagram of pipe leakage [9]]

- Previous methods for the modification of Henry-Fauske Model are not suitable for the condition of  $L/D < 3$

## Characteristics of two-phase flow in short length channel [10]

There is **no enough time to become a equilibrium condition** at the choking location.

Fluid passing will not have sufficient time to completely **nucleate before leaving the pipe or tube**.

## Non-equilibrium factor and steam quality

Reference	Non-equilibrium factor ( $N$ )	Steam quality ( $x$ )	Remark
Henry [2]	$N = 20x_{eq} \quad (x_{eq} \leq 0.05)$ $1 \quad (x_{eq} > 0.05)$	$x = Nx_{eq}\{1 - \exp[-0.0523(L/D - 12)]\}$	Applicable for $L/D > 12$
Henry and Fauske [6]	$N = x_{eq}/0.14 \quad (x_{eq} \leq 0.14)$ $1 \quad (x_{eq} > 0.14)$	$x = Nx_{eq}$	Applicable for low quality region
Xu and Wang [7]	$N = (0.037 \cdot L/D - 0.164)\exp\left(-20.7 \frac{\Delta T_{sub}}{T_c}\right)$	$x = Nx_{eq}$	Applicable for $L/D > \sim 4.34$

## Homogeneous Frozen model [11]

- The equation of sound speed of two-phase flow can be derived by combining the mass and momentum conservation of two-phase flow in one-dimension

$$a_{tp}^2 = \left\{ \left[ \alpha^2 + \alpha(1 - \alpha) \frac{\rho_f}{\rho_g} \right] \frac{d\rho_g}{dP} + \left[ (1 - \alpha)^2 + \alpha(1 - \alpha) \frac{\rho_g}{\rho_f} \right] \frac{d\rho_f}{dP} + (\rho_g - \rho_f) \frac{\alpha(1 - \alpha)}{x(1 - x)} \frac{dx}{dP} - \alpha(1 - \alpha)(\rho_g - \rho_f) \frac{dk}{dP} \right\}^{-1}$$

Polytropic, $\frac{d\rho_g}{dP} = \frac{\rho_g}{nP}$	Isentropic flow, $\frac{d\rho_f}{dP} = \frac{1}{a_f^2}$	<b>Adiabatic, <math>\frac{dx}{dP} = 0</math></b>	Homogeneous, $\frac{dk}{dP} = 0$
--	---	--	----------------------------------

$$a_{HFM}^2 = \left\{ \frac{1}{\left[ \alpha^2 + \alpha(1 - \alpha) \frac{\rho_f}{\rho_g} \right] + \left[ (1 - \alpha)^2 + \alpha(1 - \alpha) \frac{\rho_g}{\rho_f} \right] \frac{nP}{\rho_g a_f^2}} \right\} \frac{nP}{\rho_g}$$

→ Momentum equation

$$G = C_d \sqrt{\frac{2(P_0 - P_c)}{v_{f,0}}} \quad \dots (eq.3)$$

Discharge coefficient,  $C_d = \frac{\text{actual flow rate}}{\text{ideal flow rate}}$

→ Sound speed by **Homogeneous Frozen Model**

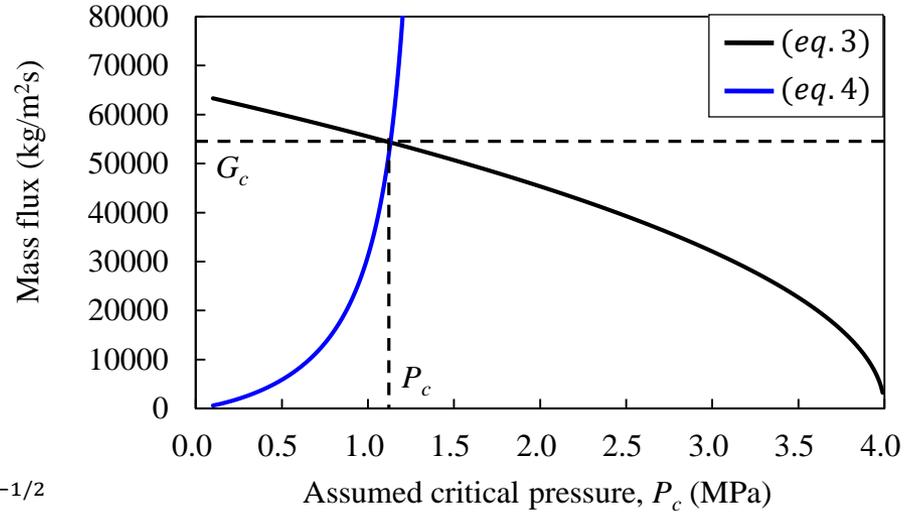
$$a_{HFM} = \left\{ \left[ \alpha^2 + \alpha(1-\alpha) \frac{\rho_f}{\rho_g} \right] + \left[ (1-\alpha)^2 + \alpha(1-\alpha) \frac{\rho_g}{\rho_f} \right] \frac{nP_c}{\rho_g a_f^2} \right\} \frac{nP_c}{\rho_g}^{-1/2}$$

Void fraction,  $\alpha = \frac{1}{1 + \frac{(1-x)\rho_g}{x\rho_f}}$

Quality,  $x = Nx_{eq}$ ,

Mixture density,  $\rho_{mix} = \alpha\rho_g + (1-\alpha)\rho_f$

Mass flux,  $G = \rho_{mix} a_{HFM} \dots (eq.4)$



Calculation condition [4]  
 [ :  $P_0=4.0$  MPa,  $\Delta T_{sub,0}=55.1$  K,  $L=8$  mm,  $D=4$  mm,  $L/D=2.0$  ]

→ Correlation of discharge coefficient, Idelchik [12]

$$C_d = \xi^{-0.5} \quad \xi : \text{total loss coefficient through sharp-edged orifice} \quad (0.015 < L/D < 2.5)$$

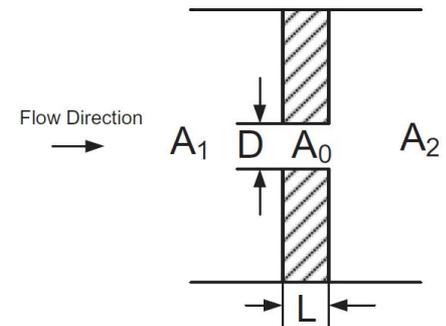
$$\xi = 0.5(1 - A_0/A_1)^{0.75} + (1 - A_0/A_2)^2 + \tau(1 - A_0/A_1)^{0.375}(1 - A_0/A_2) + fL/D$$

$$\tau = (2.4 - L/D) \times 10^{-\varphi}$$

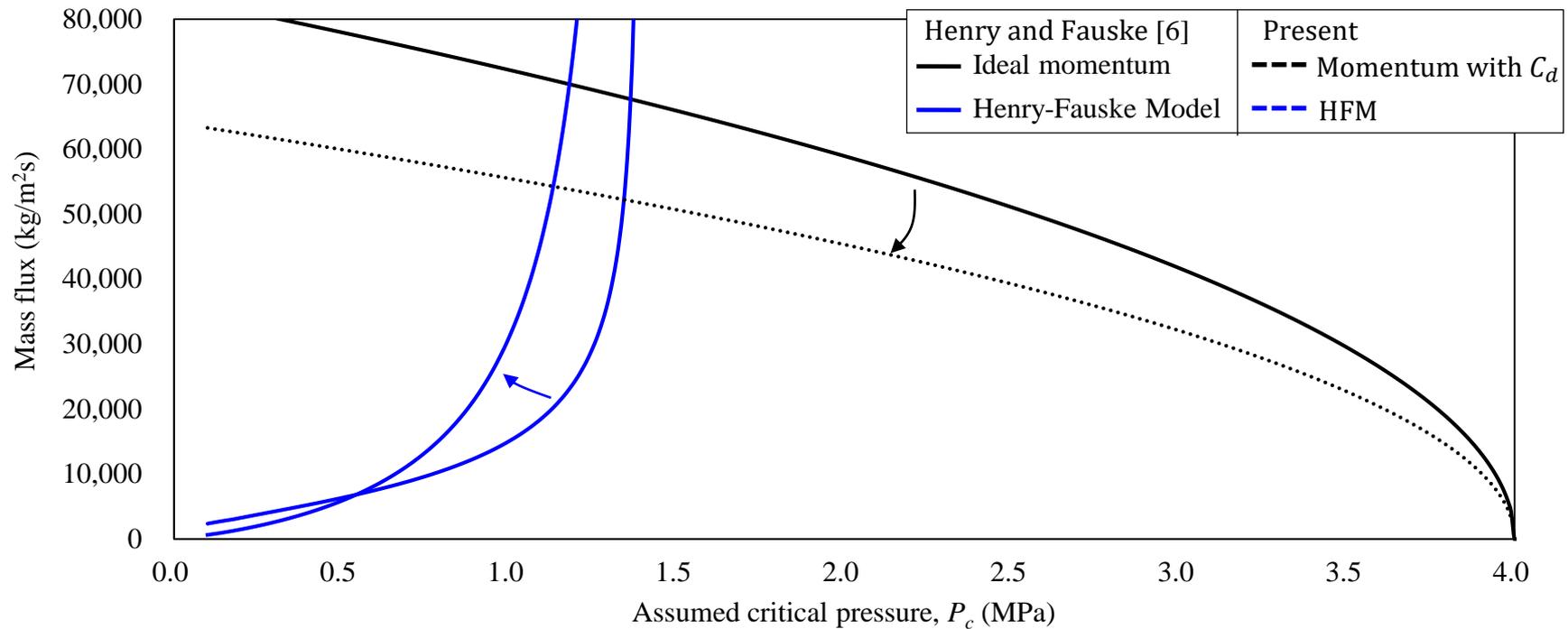
$$\varphi = 0.25 + 0.535(L/D)^8 / (0.05 + (L/D)^7)$$

$$f = 1 / (1.8 \ln(Re) - 1.64)^2$$

Filonenko and Altshul formula:  
 Friction factor for turbulent flow  
 through smooth circular tube



## Comparison of present method and Henry-Fauske Model



[ Calculation condition [4]:  $P_0 = 4.0$  MPa,  $\Delta T_{sub,0} = 55.1$  K,  $L = 8$  mm,  $D = 4$  mm,  $L/D = 2.0$  ]

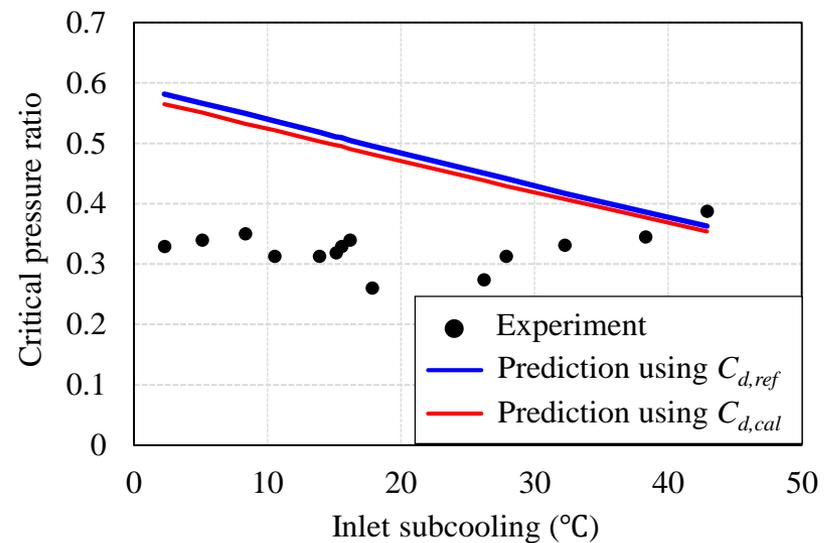
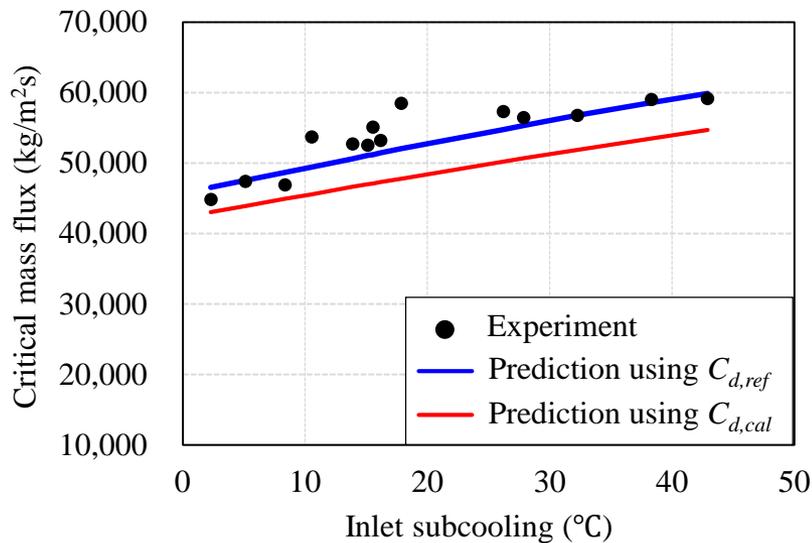
	Experiment [4]	Henry-Fauske [6]	Present
Critical mass flux (kg/m <sup>2</sup> s)	54,400	67,589	53,955
Critical pressure ratio	-	0.34	0.28
Discharge coefficient	0.77	-	0.81

- The present method shows more accurate critical mass flux than the Henry-Fauske model
- The critical pressure ratio by the present method is the same with the Fauske's investigation.

## Sozzi and Sutherland's experiments [3]

$P_0$ (MPa)	$\Delta T_{sub}$ (°C)	Geometry	$L$ (mm)	$D$ (mm)	$L/D$	$C_{d,ref}$	$C_{d,cal}$
6.5	2.3 ~ 42.9	Tube	4.7	12.7	0.37	0.73	0.66

## Comparison of prediction and experimental results [3]



- The predicted critical mass flux using  $C_{d,ref}$  shows -4.3 ~ 2.3 % difference with the measured critical mass flux using  $C_{d,cal}$  shows -10.3 % difference with the measured critical mass flux
- The measured critical pressure ratio shows almost constant values (~0.3).  
On the other hand, the calculated pressure ratio is 0.56 at low subcooling and reduced as the subcooling increases.
- Physically, the pressure ratio is inversely proportional to the mass flow rate.  
The critical pressure ratio would be difficult to measure accurately

## ● Park et al.'s experiments [4]

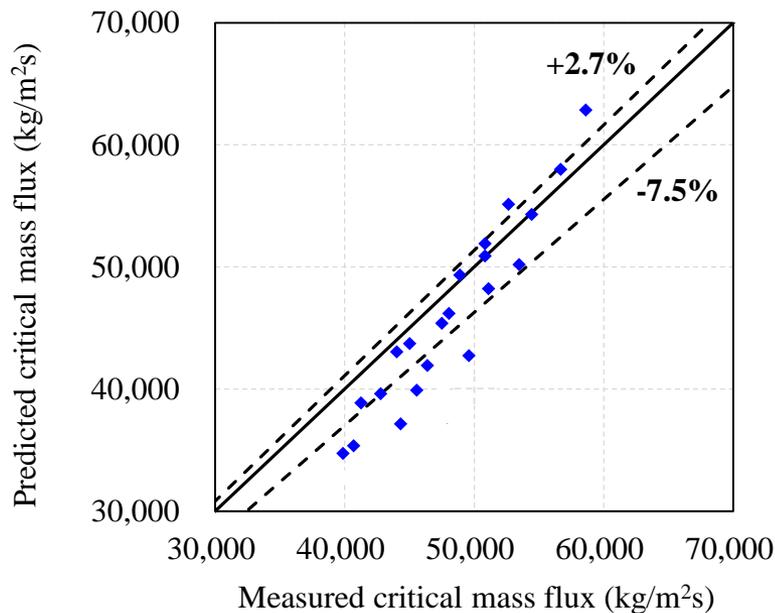
$P_0$ (MPa)	$\Delta T_{sub}$ (°C)	Geometry	$L$ (mm)	$D$ (mm)	$L/D$	$C_{d,ref}$	$C_{d,cal}$
4.0	2.0 ~ 105.6	Tube	2	4	0.5	0.67	0.66
			4	4	1	0.72	0.76
			8	4	2	0.77	0.82
			4	8	0.5	0.63	0.73
			8	8	1	0.61	0.78

$D = 4$  mm : with an increase of  $L/D$ , the difference between  $C_{d,ref}$  and  $C_{d,cal}$  increase

$D = 8$  mm : difference between  $C_{d,ref}$  and  $C_{d,cal}$  becomes higher than 0.1 this causes inaccuracy prediction of  $G_{cri}$

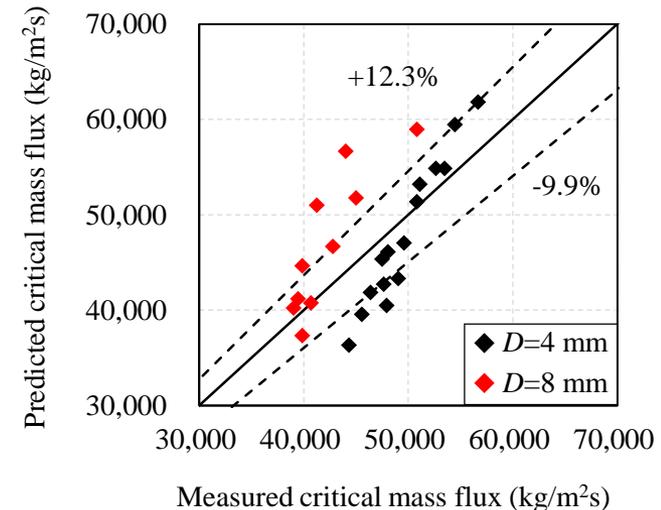
## ● Comparison of prediction and experimental results [4]

### ■ Prediction using $C_{d,ref}$



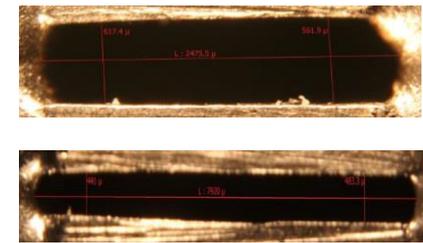
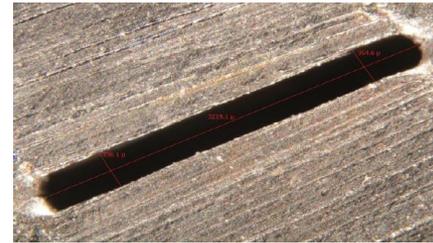
$$\text{Mean relative error : } \bar{X}_m = \frac{1}{n} \sum_{i=1}^n X_{im}, \quad X_{im} = \frac{G_{c,pre} - G_{c,exp}}{G_{c,exp}}$$

### ■ Prediction using $C_{d,cal}$



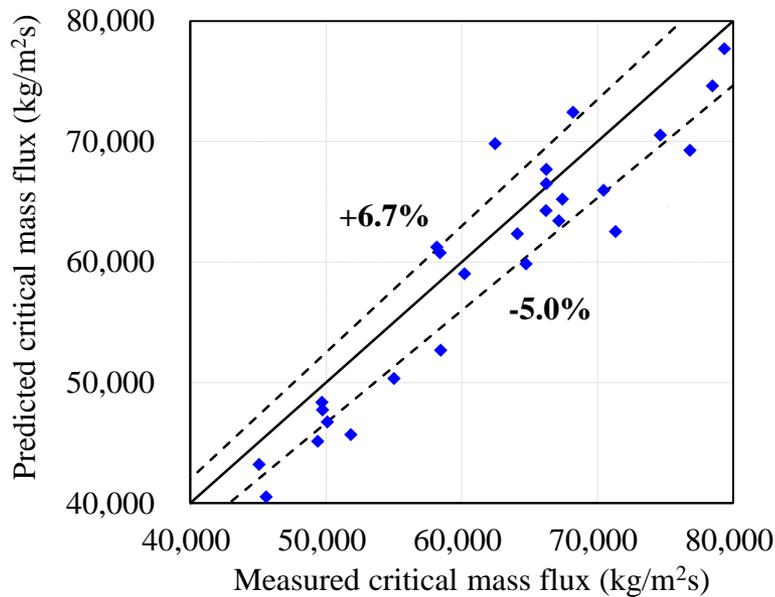
## ● Revankar et al.'s experiments [1]

$P_0$ (MPa)	$\Delta T_{sub}$ (°C)	Geometry	$L$ (mm)	$D$ (mm)	$L/D$	$C_{d,ref}$
6.8	14~ 51	Slit	1.3	0.62 ~ 0.98	1.33 ~ 2.11	0.54~ 0.9

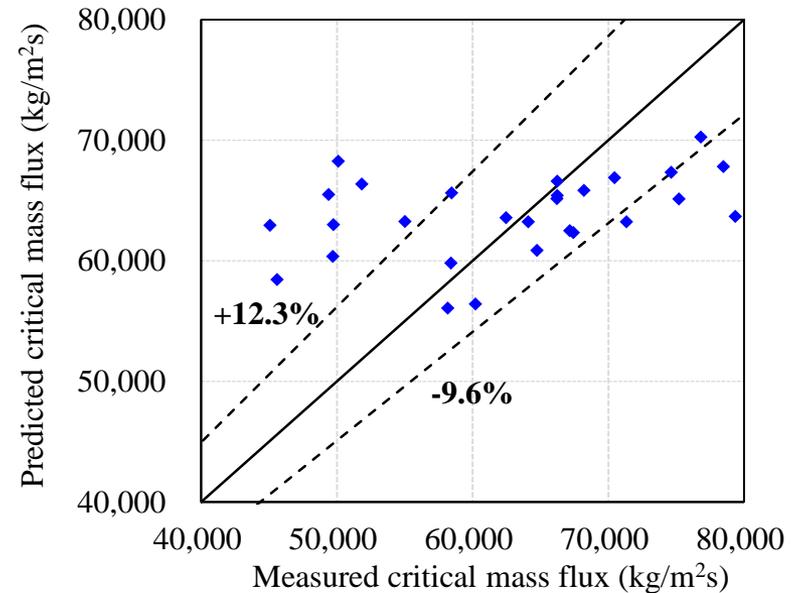


## ● Comparison of prediction and experimental results [slit]

### ■ Prediction using $C_{d,ref}$



### ■ Prediction using $C_{d,cal}$



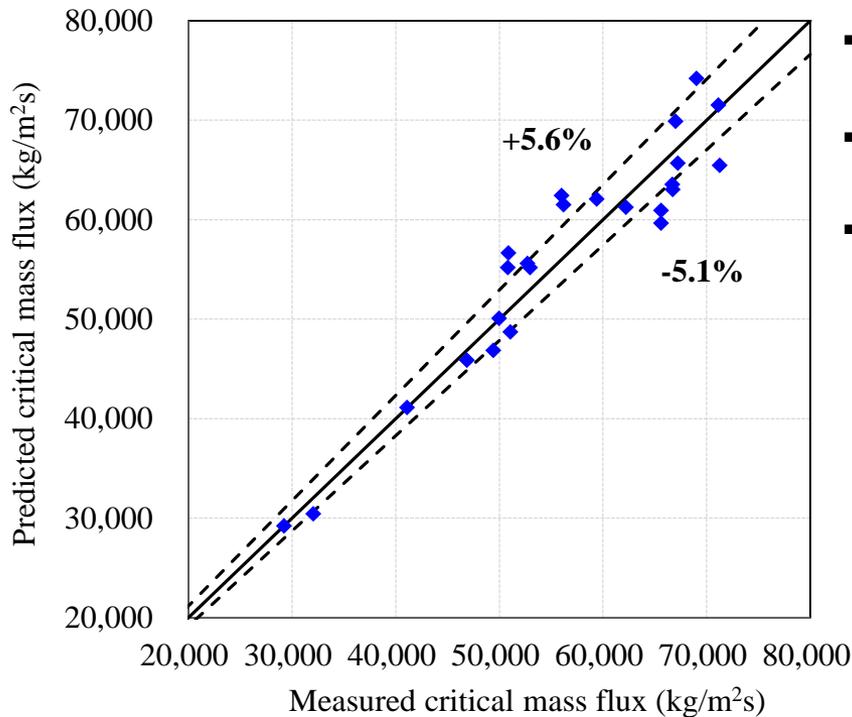
- $C_{d,cal}$  is in the range of 0.79~0.81. However, for some cases  $C_{d,ref}$  is 0.54 and 0.61.
- For slit, the geometry effects such as aspect ratio should be considered in the calculation of  $C_{d,cal}$

● Revankar et al.'s experiments [5]



$P_0$ (MPa)	$\Delta T_{sub}$ (°C)	Geometry	$L$ (mm)	$D$ (mm)	$L/D$	$C_{d,ref}$
1.3~6.89	12 ~ 55	Crack	1.2	0.45, 0.65	1.85, 2.67	-

● Comparison of prediction and experimental results [crack]



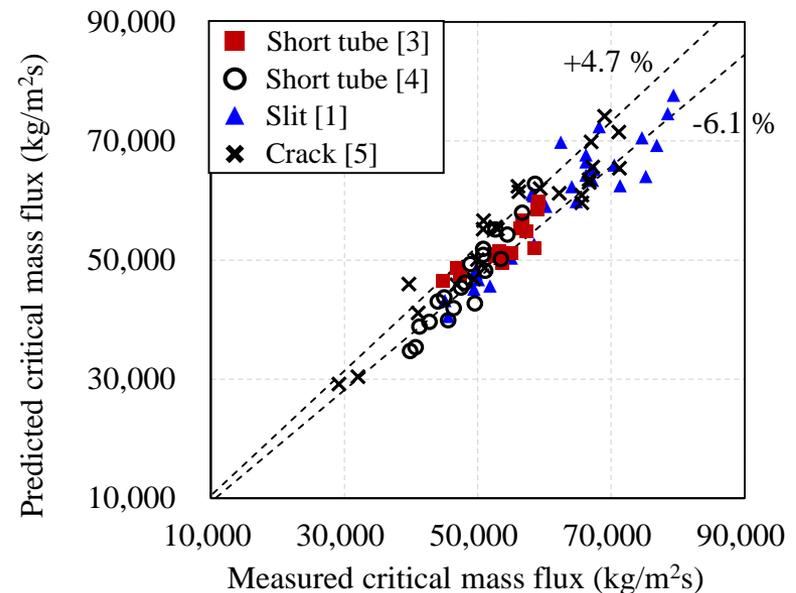
- Since  $C_{d,ref}$  was not informed in the reference [5], the critical mass flux is predicted based on  $C_{d,cal}$  by Idelchik correlation.
- As mentioned before, the correlation of Idelchik has limitations to use for the geometries which are not circular shape.
- However, the predicted mass flux shows good agreement with experimental results

- Based on the characteristics of two-phase critical flow through short channel, **HFM** and **Mass flux calculation with  $C_{d,ref}$**  are used to predict the critical mass flux.
- The present method shows good agreement in various geometries and the existing correlations for short channel are not applicable for various geometries

[ Mean relative error of present method and existing correlations compared to experimental data ]

Prediction \ Experimental data	Present method	Park et al. [4] (Correlation)	Revankar et al. [1] (Correlation)
Sozzi & Sutherland [3](short tube)	-4.3 ~ 2.3 %	17.3 %	-20.4 ~ 8.4 %
Park et al. [4] (short tube)	-7.5 ~ 2.7 %	-1.3 ~ 2.5 %	-3.2 ~ 29.7 %
Revankar et al. [1] (slit)	-6.7 ~ 5.0 %	-2.3 ~ 11.5 %	-8.5 ~ 25.4 %
Revankar et al. [5] (crack)	-5.1 ~ 5.6 %	15.6 %	-16.3 ~ 13.0 %

- The present method can be applied for the short length channel with various cross-sections in the range of  $L/D < 3$ , and  $\Delta T_{sub} \leq 100^\circ\text{C}$ .
- For some cases, the results of **critical mass flux using calculated  $C_d$**  are not comparable with experiments. It is **necessary to consider the effects of diameter and cross-section shape**.



- [1] S.T.Revankar, B. Wolf, and A. Vadlamnai, Assessment of leak rates through steam generator tubes, PU/NE-13-11, Pur-due University, 2013.
- [2] R.E. Henry, The two-phase critical discharge of initially saturated or subcooled liquid, Nuclear Science and Engineering, Vol. 41, pp. 336-342, 1970.
- [3] G.L. Sozzi, W.A. Sutherland, Critical flow of saturated and subcooled water at high pressure, NEDO-13418, General Electric Company, 1975.
- [4] C.K. Park, S. Cho, T.S. Kwon, S.K. Yang, M.K. Chung, An experimental investigation of maximum flow rates of subcooled water through square edge orifices with small diameters, NTHAS2: Second Japan-Korea Symposium on Nuclear Thermal Hydraulics and Safety, Fukuoka, Japan, Oct. 15-18, 2000.
- [5] S.T. Revankar, J. Riznic, An experimental investigation of subcooled choked flow in actual steam generator tube cracks, Nuclear Engineering and Design, Vol. 354, 110144, 2019.
- [6] R.E. Henry, H.K. Fauske, The two-phase critical flow of one-component mixtures in nozzles, orifices, and short tubes, Journal of Heat Transfer, Vol.93, pp.179-187, 1971.
- [7] J. Xu, R. Wang, Critical flow with high pressure water flow-ing in small diameter sharp-edged tubes, Heat and Mass Trans-fer, Vol. 35, pp. 205-211, 1999.
- [8] M. Wang, S. Qiu, G. Su, W. Tian, Research on the leak-rate characteristics of leak-before-break (LBB) in pressurized water reactor, Appl. Therm. Eng, Vol. 62, pp. 33-140, 2014.
- [9] B. Ghosh, S.K. Bandyopadhyay, S.K. Gupta, H.S. Kushwaha, V. Venat Raj, Leak rates through cracks and slits in PHT pipes for LBB. Nucl. Eng. Des. Vol. 212, pp. 85-97, 2002.
- [10] Y.S. Kim, Overview of geometric effects on the critical flow rate of subcooled and saturated water, Annals of Nuclear Energy, Vol. 76, pp. 12-18, 2015.
- [11] R.E. Henry, M.A. Grolmes, H.K. Fauske, Pressure-pulse propagation in two-phase one- and two-component mixtures, ANL-7792, Argonne National Laboratory, 1971.
- [12] I.E. Idelchik, 2008. Handbook of Hydraulic Resistance, 4th ed. Begell House, Inc.

# Q & A