An Improved Deterministic Truncation of Monte Carlo Solutions for Nuclear Reactor Analysis



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Monte Carlo (MC) calculation for high-fidelity reactor criticality analysis

- A stochastic method to solve a statistical problem finding out the average behavior of the unknown parameters based on probabilistic inference
 - Simulation of individual particle based on stochastic random sampling
 - Calculation of reactor parameters based on statistical treatment (i.e. average and variance)

$$S^{i+1} = \frac{1}{k_{eff}^{i}} HS^{i} \qquad \Leftrightarrow \qquad (L+T-S)\psi^{i+1} = \frac{1}{k_{eff}^{i}} F\psi^{i} \qquad (1)$$

where $k_{eff}^{i} = \frac{\text{fission rate at iteration } i}{\text{fission rate at iteration } i-1} = \frac{\text{fission rate at iteration } i}{\text{loss rate at iteration } i}$
 ψ : neutron angular flux $\vec{r} = (x, y, z)$: position vector
 S : neutron source H : transport operator $\hat{\Omega} = (u, v, w)$: direction vector
 $L\psi = \hat{\Omega} \cdot \nabla \psi(\vec{r}, \hat{\Omega}, E)$ (leakage loss term)
 $T\psi = \sigma_{i}(\vec{r}, \hat{\Omega}, E)\psi(\vec{r}, \hat{\Omega}, E)$ (collision loss term)
 $S\psi = \int d\hat{\Omega}' \int dE' \sigma_{s}(\vec{r}, \hat{\Omega}', E')\psi(\vec{r}, \hat{\Omega}', E')$ (scattering production term)
 $F\psi = \int d\hat{\Omega}' \int dE' v\sigma_{f}(\vec{r}, \hat{\Omega}', E')\psi(\vec{r}, \hat{\Omega}', E')$ (fission production term)



Monte Carlo (MC) calculation for high-fidelity reactor criticality analysis

Pros

High accuracy

- Direct simulation of particles' whole behavior
- No discretization of variables (energy, angle, space)
- No constraints on geometry construction
- High parallelization
- Parallel calculation of individual particles

Cons

- Computationally expensive
- Large memory to describe explicit geometry and to utilize cross section data
- Long time to obtain the converged source distribution and to get quantities of interest
- Uncertainties and inconsistency
- Stochastic uncertainties
- Underestimation of variance
- Fundamental dilemma
- The main calculation is activated when the FSD converges
- Several studies have been conducted to accelerate the calculation speed and to reduce stochastic uncertainties more efficiently
 - Coarse mesh finite difference (CMFD) method
 - Modified power method
 - ...



Overview

- A statistic treatment of deterministic solutions determined by FMFD-assisted MC
 - To accelerate the convergence of the fission source distribution by adjusting particles' weight
 - To provide a subset of solutions to the original MC approach





Improved DTMC in a MC simulation



- ✓ p-CMFD (partial current based coarse mesh finite difference)
- ✓ p-FMFD (partial current based fine mesh finite difference)
- ✓ iDTMC (improved deterministic truncation of MC solution)



DTMC

Deterministic truncation of MC solution method

- A conventional numerical scheme of the DTMC method
 - FMFD has been applied throughout the simulation for both acceleration and variance reduction
 - Instability and inconsistency problems





iDTMC

Improved deterministic truncation of MC solution method

- A numerical strategy has been applied by the p-CMFD and p-FMFD in a combined way
 - p-CMFD : stable and consistent deterministic calculation, and more efficient than the p-FMFD
 - **p-FMFD** : reliable detailed reaction solutions





iDTMC

Improved deterministic truncation of MC solution method

Single cycle coupled p-CMFD

- A stable and consistent deterministic calculation is available even without the cycle accumulation
- The p-CMFD enables the fast convergence of the FSD
- Cycle-cumulative decoupled p-FMFD
 - A stable and reliable deterministic solutions can be obtained with long cycle accumulation
 - The only reliable deterministic solutions are obtained from the p-FMFD method
 - · Coupling is not necessary because the FSD already converged





CMFD & FMFD

Coarse mesh finite difference (CMFD) method

- Solving the lower-order diffusion-like equation with the surface current correction
 - Fast and efficient deterministic calculation
 - MC-equivalent accuracy based on the generalized equivalent theory (GET)
- Unavailable to produce the detailed power distribution \rightarrow radial direction : assembly size (~ 20 cm)

Fine mesh finite difference (FMFD) method

- Fine mesh grid to generate the detailed pin-wise power distribution
 - Radial direction : pin size (~ 1 cm)
 - Axial direction : 10 15 cm



p-FMFD

Partial current based fine mesh finite difference (p-FMFD) method

Neutron balance equation (diffusion-like one-group deterministic equation)

$$\sum_{s=x,y,z} \frac{a_s}{v_i} ((j_{s_1}^+ - j_{s_1}^+) - (j_{s_0}^+ - j_{s_0}^-)) + \sigma_a^i \phi_i = \overline{s_i}$$
(2)
where j^{\pm} : partial current
 σ_a : absorption cross section
 $v\sigma_f$: no. of fission neutrons × fission XS
 ϕ : neutron flux
 a : surface area
 v : node volume
 s : surface index $(s_1 = i + 1/2 \text{ and } s_0 = i - 1/2)$
 i : node index
 $\overline{s_i} = \frac{1}{k} v \sigma_f^i \phi_i$: fission source
 $j_{i+1/2}$ $j_{i+1/2}$ $j_{i+1/2}$
 $s_0 = i - \frac{1}{2}$ $s_1 = i + \frac{1}{2}$
(2)



One node p-CMFD

One-node CMFD acceleration

- 1-CMFD scheme is applied to accelerate the FMFD deterministic calculation
- Coarse mesh grid
 - Radial direction : assembly size (~ 20 cm)
 - Axial direction : 20 30 cm



- Fast and efficient calculation
- High parallelism



Methods :

Stochastic error in MC calculation

- Stochastic error cannot be exactly estimated with a single MC run
 - Apparent standard deviation (SD) is underestimated due to a cycle correlation
 - Variance underestimation is more critical issue in iDTMC method because of correlation of the cycles and parameters



Fission reaction in current generation → Neutron source in next generation



Error quantification of iDTMC method





Error quantification of iDTMC method

- Flow chart





Error quantification of iDTMC method

FMFD parameters to calculate k_{eff}

• Group constants (cross sections) are calculated from MC simulation every cycle





Random number generation

- Latin hypercube sampling (LHS)
 - A statistical method for generating a near random sample of parameter values from a multidimensional distribution
 - An efficient random sampling by analyzing variable space

how many sample points to use

in which row and column the sample point was taken

• More evenly and fairly distributed for the limited sample size



Random sampling



LHS sampling



Correlation sampling

- Correlation matrix
 - Correlation between total, absorption and nu X fission cross section
 - Correlation coefficient can be calculated by

$$o_{X,Y} = \frac{\operatorname{cov}(X,Y)}{\sigma_X \sigma_Y}$$

• Correlation matrix can be composed as the follows

$$\mathbf{C} = \begin{array}{c|c} \Sigma_t & \Sigma_a & V\Sigma_f \\ \hline \Sigma_t & \rho_{t,t} & \rho_{t,a} & \rho_{t,f} \\ \hline \Sigma_a & \rho_{a,t} & \rho_{a,a} & \rho_{a,t} \\ \hline V\Sigma_f & \rho_{f,t} & \rho_{f,a} & \rho_{f,f} \end{array}$$

- Cholesky decomposition
 - Correlation matrix C (positive-definite) is decomposed to be a form of

$$\mathbf{C} = \mathbf{L}\mathbf{L}^T$$

· Positive definiteness should be improved for pseudo non-positive definite due to stochastic uncertainty

$$\mathbf{C'} = \mathbf{C} + 0.1 \times I$$



Correlation sampling

- Conversion by inverse cumulative density function (CDF)
 - Underlying function is assumed to be a normal distribution
 - Random parameters can be obtained by the uniform random number (URN) calculated from LHS

$$p = \sqrt{2} erf^{-1}(\gamma) \in P^{3 \times N}$$

where erf : error function

 γ : URN \in [0,1)

Correlated parameters generation

• Correlated parameters can be generated by multiplying the Cholesky-decomposed lower triangular matrix and the matrix for random parameters calculated by LHS

$$\mathbf{G} = \mathbf{L} \times \mathbf{P}$$
$$(3 \times N_s) = (3 \times 3) \times (3 \times N_s)$$

Correlated URN

- Correlated parameters are again converted to be URNs
- Correlated URNs can be obtained by the CDF conversion

$$\gamma' = \frac{1}{2} \left(1 + erf\left(\frac{g}{\sqrt{2}}\right) \right)$$

where g: element of matrix **G**



Correlation sampling

Correlated cross section sampling

- Using the correlated URNs, the correlated cross sections can be sampled
- In the sampling of the cross section, the probability function is created by the FMFD parameters
 - → PDF made from the FMFD parameters does not follow the normal distribution
- The cross sections are directly sampled from the actual given PDF



Eigenvalue calculation by 1st order perturbation theory

- 1st order perturbation theory
 - Multiplication factor can be easily calculated with the perturbed parameters



- Forward and adjoint fluxes are different in the p-FMFD method due to the correction factors
- But they are comparable each other with some reasons
- Therefore, the self-adjointness is assumed in 1st PT error quantification



Core configuration

- A small modular reactor problem
- 7 X 7 fuel assemblies surrounded by a water reflector





Calculation condition

- Total 112 cores of Xeon E5-2697 with clock speed of 2.60 GHz
- Skip p-CMFD : 1
- Skip early cycles : 5
- According to SCI
 - Minimum generation size = 6,000,000 histories per cycle

Total number of fine nodes		No. of neutrons		Off-peaking		Optimum generation size	
213,860	×	5.86	/	0.2 =	6,266,098	≅	6,000,000

- The number of inactive cycles were automatically determined
- 10 active cycles were used
- For real standard deviation, 20 independent runs simulated with different random seeds
- Reference solution : 1.27774 ± 1.2 pcm
 - No. of histories : 6,000,000
 - No. of inactive cycles 120
 - No. of active cycles : 500
 - No. of batches : 2

FMFD parameters

- Three pin positions are arbitrarily selected to characterize the FMFD parameters





FMFD parameters

- Convergence behavior; central region (62,59,10)



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FMFD parameters

- Convergence behavior; peripheral region (52,24,10)





FMFD parameters





FMFD parameters





1st PT vs. direct calculation

- The number of samples



Computing time of 1st PT vs. direct calculation for 100 samples

	1st PT	Direct
Time (sec.)	4.7	158.8



FSD convergence

- By the Shannon entropy





Multiplication factor & stochastic errors

Danamatan			Cycle						
Fara	meter	1	3	10	15	20			
	$k_{e\!f\!f}$	1.27706	1.27763	1.27781	1.27782	1.27778			
MC-CMFD	σ_{a}	-	18.3	13.0	9.9	8.9			
	$\sigma_{\rm r}$	46.4	24.2	13.7	13.4	11.3			
	$k_{e\!f\!f}$	1.27777	1.27776	1.27776	1.27775	1.27775			
DTMC	σ_{a}	-	Cycle131015277061.277631.277811.27782-18.313.09.946.424.213.713.4277771.277761.277761.27775-0.60.50.55.75.75.24.96.46.66.05.5	0.5					
IDIMC	Parameter Cycle 1 3 10 15 k_{eff} 1.27706 1.27763 1.27781 1.277 σ_a - 18.3 13.0 9.9 σ_r 46.4 24.2 13.7 13.2 k_{eff} 1.27777 1.27776 1.27776 1.2777 σ_a - 0.6 0.5 0.5 σ_r 5.7 5.7 5.2 4.9 1st PT 6.4 6.6 6.0 5.5	4.9	4.8						
	1 st PT	6.4	6.6	6.0	5.5	5.2			

* $k_{eff}^{ref} = 1.27774 \pm 1.2 \text{ pcm}$



Real standard deviation of the multiplication factor



Much more reliable solutions can be obtained with the iDTMC method Remind that the iDTMC method is designed to pursue the early truncation



Comparison of standard deviation

- iDTMC vs. 1st PT



They show great agreement each other throughout the simulation The reliable stochastic error can be calculated with a single batch calculation











real standard deviation of the 2D phi	howei -	Cycle 10	iDTMC	Direct
At cycle 10	-	Avg.	0.009	0.009
			1000 1000	
Martin within within worth within Nitra Score worth worth within	81114 1111			
	2003 2002			

iDTMC

Parameter sampling

 $\sigma_{i,j} = \frac{1}{N_b} \sum_{b}^{N_b} (p_{i,j}^b - \overline{p}_{i,j})^2 \text{ where } b : \text{batch number, } N_b : \text{No. of batches } (40)$



Relative error distribution for the 2D pin po	ower	Error (%)	MC-CMFD	iDTMC	
		Avg.	3.2	0.8	_
- Al Cycle I		Max.	32.2	7.5	_ (%)
	annach S				- 30
					- 25
					- 20
					- 15
					10
	72591			62	5
MC-CMFD		iDTI	ИС	_	-
$arepsilon_{i,j} = \left rac{p_{i,j} - p_{i,j}^*}{p_{i,j}^*} ight imes$:100 (%)	where p^* : refere	nce pin power	(41)
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Relative error distribution for the 2D pin	power Error (%)	MC-CMFD	iDTMC	_
At evolo 10	Avg.	1.0	0.6	_
- Al Cycle IU	Max.	9.6	5.6	_ (%)
			142	- 30
				- 25
			10.20	- 20
				- 15
				- 10
			12	- 5
MC-CMFD	iDT	MC		
$arepsilon_{i,j} = \left rac{p_{i,j} - p_{i,j}^*}{p_{i,j}^*} ight $	$\frac{j}{2} \times 100 \ (\%)$ where p^* : refer	ence pin power	(42	<i>.</i>)



Computing time

Deterministic calculation

Mathada		p-FMFD	
Methods	р-сигр	w/o one-node p-CMFD	w/ one-node p-CMFD
Time (sec.)	0.03	97.9	1.4

MC calculation

	Standard MC	MC-CMFD	iDTMC
No. of inactive cycles	81	23.1	23.1
No. of active cycles	10	10	10
Inactive time (hr.)	1.2	0.7	0.8
Active time (hr.)	0.29	0.29	0.51
Total time (hr.)	1.47	0.95	1.29

p-FMFD for solution prediction Variance estimation



FOM for the multiplication factor





APR1400 quarter core problem

- 1st cycle fuel loading pattern
- 241 fuel assemblies
- Fuel zoning & Bas are modelled





Axial configuration

Radial configuration



Calculation condition

- Total 112 cores of Xeon E5-2697 with clock speed of 2.60 GHz
- Skip p-CMFD : 1
- Skip early cycles : 5
- According to the SCI
 - Minimum generation size = 6,000,000 histories per cycle

Total number of fine nodes	No. of neutrons	Off-peaking	Optimum generation size		
586,112	× 5.86	/ 0.2 = 17,173,081	≅ 20,000,000		

- The number of inactive cycles were automatically determined
- 10 active cycles were used
- For real standard deviation, 30 independent runs simulated with different random seeds
- Reference solution : 1.20392 ± 0.82 pcm
 - No. of histories : 10,000,000
 - No. of inactive cycles 60
 - No. of active cycles : 300
 - No. of batches : 4

FSD convergence behavior



Much faster source convergence is achieved in the big size reactor problem which has a higher dominance ratio compared to the standard MC



Multiplication factor & stochastic errors

Parameter		Cycle					
		1	3	10	15	20	
	$k_{e\!f\!f}$	1.20390	1.20392	1.20391	1.20394	1.20390	
MC-CMFD	σ_{a}	-	10.8	10.0	6.5	-	
	$\sigma_{\rm r}$	16.6	8.8	7.1	5.2	16.6	
	$k_{e\!f\!f}$	r 1 k_{eff} 1.20390 1 σ_a - - σ_r 16.6 - k_{eff} 1.20392 1 σ_a - - σ_r 2.9 1 1st PT 3.2 -	1.20392	1.20389	1.20391	1.20392	
DTMC	σ_{a}	-	0.4	0.4	0.4	-	
IDIMC	$\sigma_{\rm r}$	2.9	2.9	2.8	2.0	2.9	
	1 st PT	3.2	2.8	2.7	2.8	3.2	

* $k_{eff}^{ref} = 1.20392 \pm 0.82 \text{ pcm}$



Real standard deviation of the multiplication factor



Much more reliable solutions are obtained with the iDTMC method compared to the CMFD Consistent results are shown in the different types of the problem







iDTMC

$$\sigma_{i,j} = \frac{1}{N_b} \sum_{b}^{N_b} (p_{i,j}^b - \overline{p}_{i,j})^2 \text{ where } b \text{ : batch number, } N_b \text{ : No. of batches } (44)$$



Real standard deviation of the 2D pin power Cycle 10 MC-CMFD **iDTMC** At cycle 10 0.005 0.013 Avg. 0.1 0.09 0.08 0.07 100 C 100 Sec. 18. 0.06 STREET, NO. NO. OF STREET, NO. 0.05 ALC: N. MARCHINE, M. M. 0.04 0.03 70-20 50 0.02 100 0.01

MC-CMFD

iDTMC

$$\sigma_{i,j} = \frac{1}{N_b} \sum_{b}^{N_b} (p_{i,j}^b - \overline{p}_{i,j})^2 \text{ where } b \text{ : batch number, } N_b \text{ : No. of batches } (45)$$



Polative error distribution for the 2D pin power -	Error (%)	MC-CMFD	iDTMC	
Relative error distribution for the 2D pin power	Avg.	2.3	0.42	
– At cycle 1	Max.	22.3	3.8	(%)
	128.201		343	- 22
				- 20
			14	- 18
				- 16
				- 14
				- 12
				- 10
				- 8
				- 6
				- 4
				- 2
MC-CMFD	iDTMC			
$\varepsilon_{i,j} = \left \frac{p_{i,j} - p_{i,j}^*}{p_{i,j}^*} \right \times 100 \ (\%)$	where p^* : refere	nce pin power	(4	6)
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Polative error distribution for the 2D pin now	Err	or (%)	MC-CMFD	iDTMC	
Relative error distribution for the 2D pin pow		Avg.	0.8	0.4	
 At cycle 10 	N	Лах.	7.5	3.5	(%)
	12/2/2012	67.77 P.		'nf"	22
					- 20
					- 18
					- 16
					- 14
					- 12
					- 10
					- 8
					- 6
					- 4
					- 2
MC-CMFD	iC	DTMC			
$\varepsilon_{i,j} = \left \frac{p_{i,j} - p_{i,j}^*}{p_{i,j}^*} \right \times 100$)(%) wher	$p = p^*$: referen	ce pin power	(4	7)
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Computing time

MC calculation

	Standard MC	MC-CMFD	iDTMC
No. of inactive cycles	160	24.4	25.5
No. of active cycles	10	10	10
Inactive time (hr.)	14.0	2.4	2.4
Active time (hr.)	0.9	0.9	1.0
Total time (hr.)	15.0	3.3	3.4



FOM for the multiplication factor



Much higher numerical performance is achieved with the iDTMC method over 5 to 30 times higher



Conclusions

Conclusions

- The iDTMC method has been developed for efficient neutornic reactor analysis
 - Potential bias and numerical instability disappear.
 - The convergence of the FSD is accelerated and thus the computing time is reduced.
 - The stochastic uncertainty is decreased even from the beginning of the active cycle.
 - The stochastic error can be reasonably measured by parameter sampling scheme.
 - The numerical performance is enhanced by the comparison of the conventional CMFD-assisted MC method.

Future work

- Various applications
 - Burn up calculation
 - Multi-physics calculation
 - Fast reactor analysis



Thank you for your attention