

Transactions of the Korean Nuclear Society Autumn Meeting
Online, December 16-18, 2020

Preliminary analysis of iodine behavior in ISLOCA

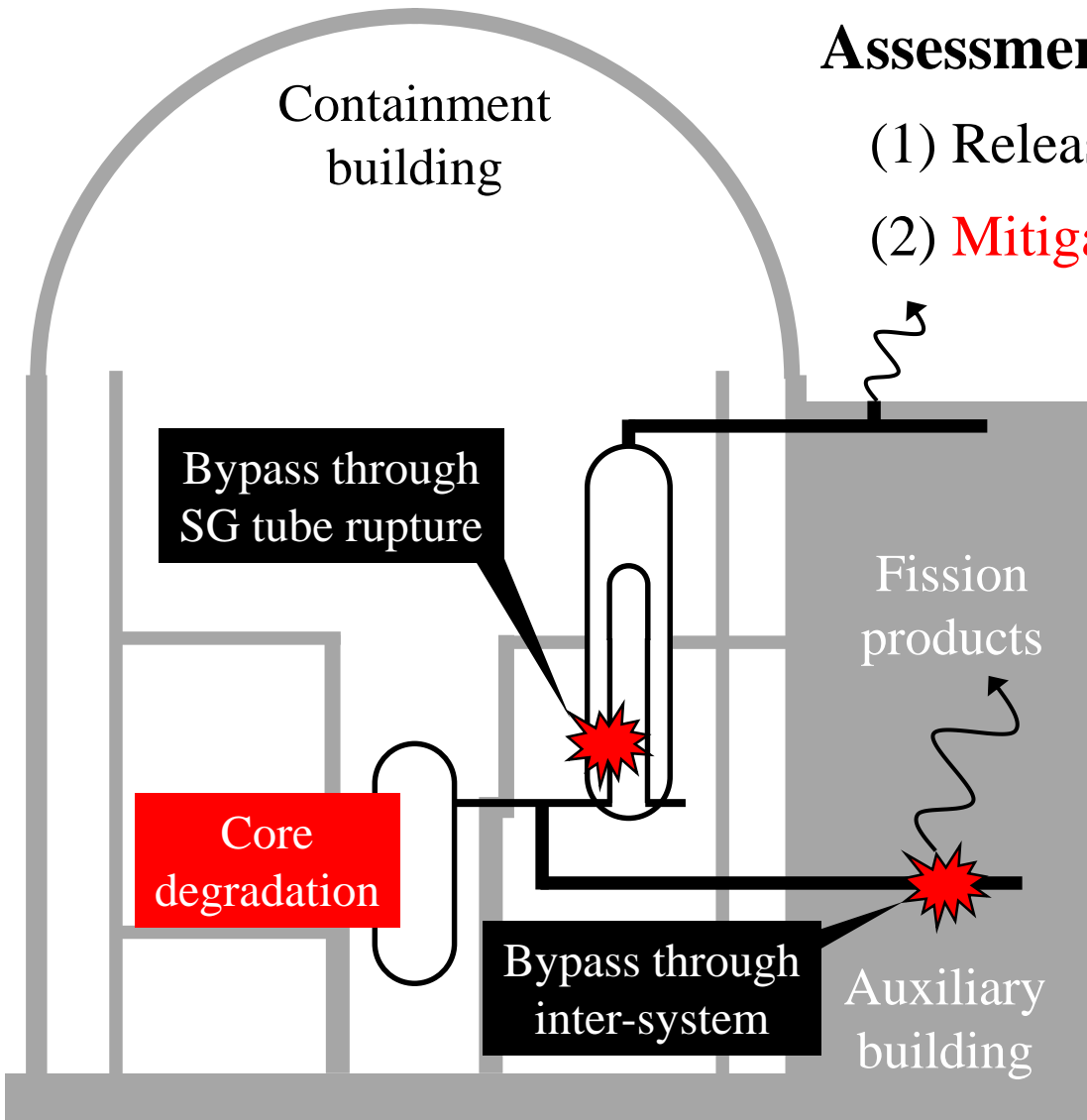
Activation of iodine pool chemistry model in MELCOR(Ver. 2.2)

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Bypass accident



Assessment of accident source term

- (1) Release rate of radioactive nuclide
- (2) **Mitigation effect** during release

1962 TID-14844

Experts' opinions
(Conservative approximation)

1995 NUREG-1465

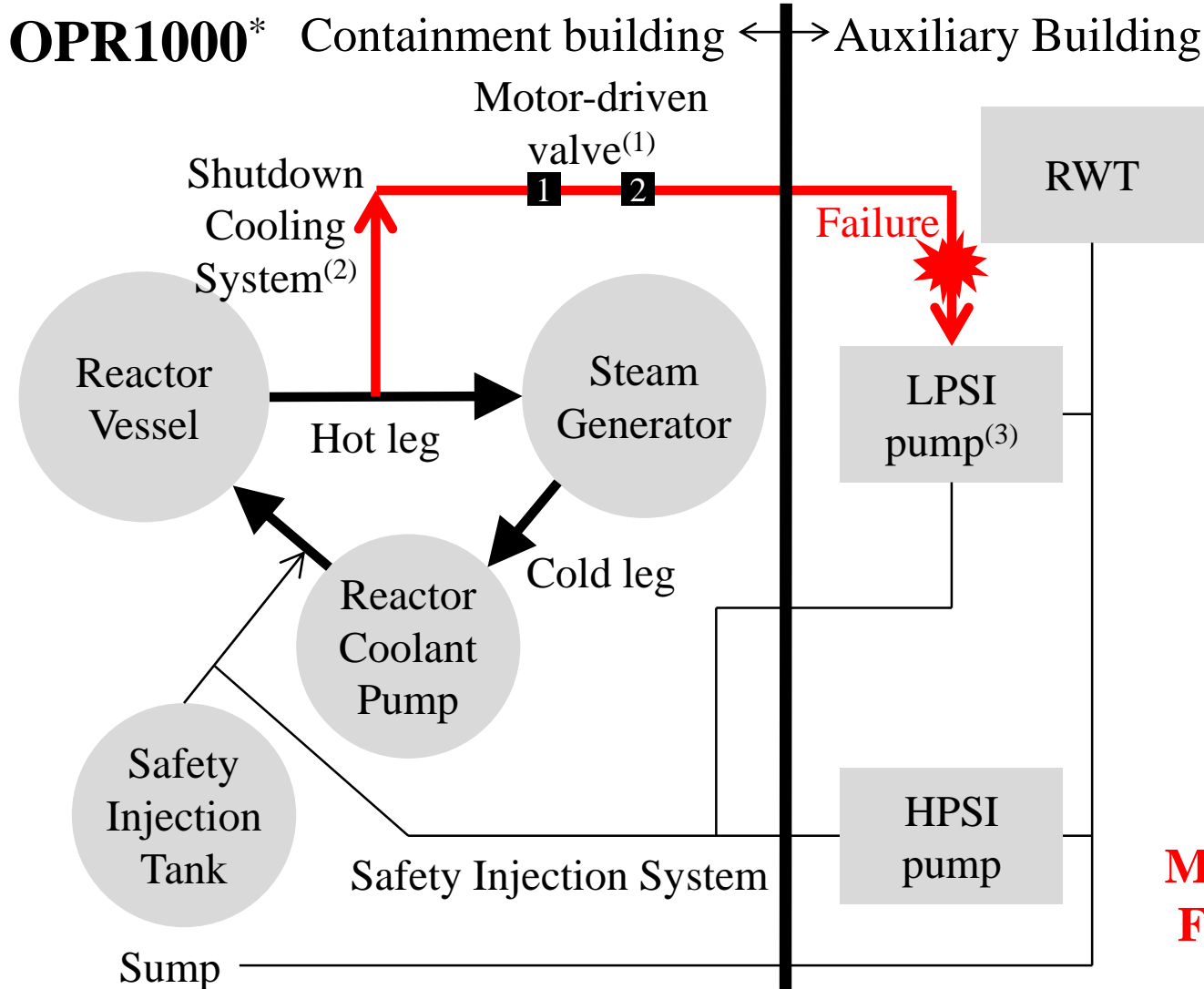
Code for severe accident
(Experiments + phenomena)

2012 SOARCA

Code with detailed models

**Best-estimated source term
of ISLOCA**

ISLOCA scenario



Sequence

- (1) MOV failure**
- (2) Coolant flows through SCS pipe
- (3) LPSI pump front is ruptured
- (4) Core dry → SA
- (5) Fission products discharge into AB

Modeling of behavior of FP in MELCOR(v 2.2)

*MELCOR input from KINS/HR-464/2002, **KAERI/TR-6020/2015

Iodine behavior in ISLOCA

1 CsI aerosol dynamics

generation

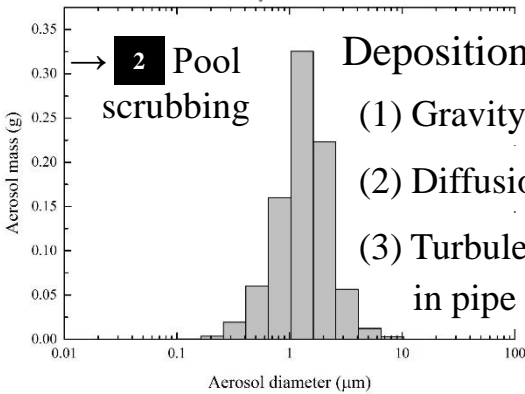
gas↔particle

Size(mass) distribution(t) =
Source ± Condensation/Evaporation
± Agglomeration - Deposition

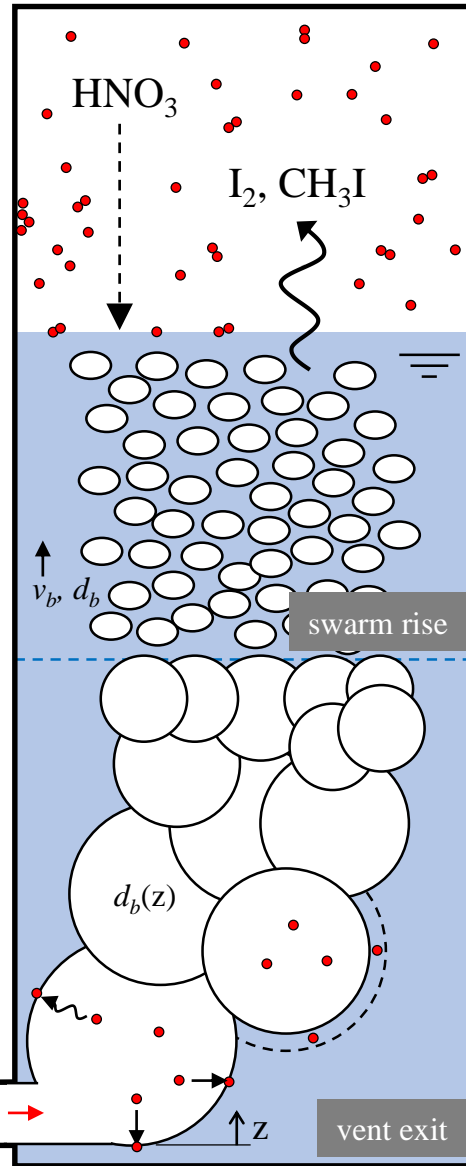
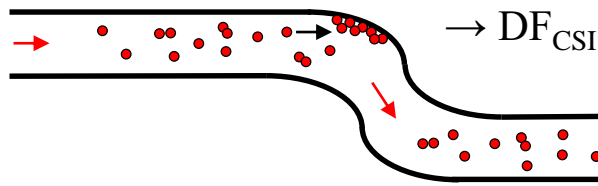
combination

MAEROS code

removal



Deposited aerosol on SCS pipelines



3 Iodine pool chemistry

$$[I_{2,atm}] = f([I_{2,aq}], PC_{I_2}(T))$$

Radiolysis, reaction = $f(\text{pH, dose rate})$ ↑ Chemical eqs. ~#200
 $A + B \rightleftharpoons C + D$

I, OH⁻, H⁺... in late pool

2 Pool scrubbing in AB

SPARC-90 code

vent exit

swarm rise

$$DF = DF_1 \times DF_2 \times DF_3 \times DF_4$$

condensation

inertial
impaction(d_p)

gravity(d_p),
diffusion,
centrifugal

condensation,
gravity,
diffusion,
centrifugal

$$= f(d_p)$$

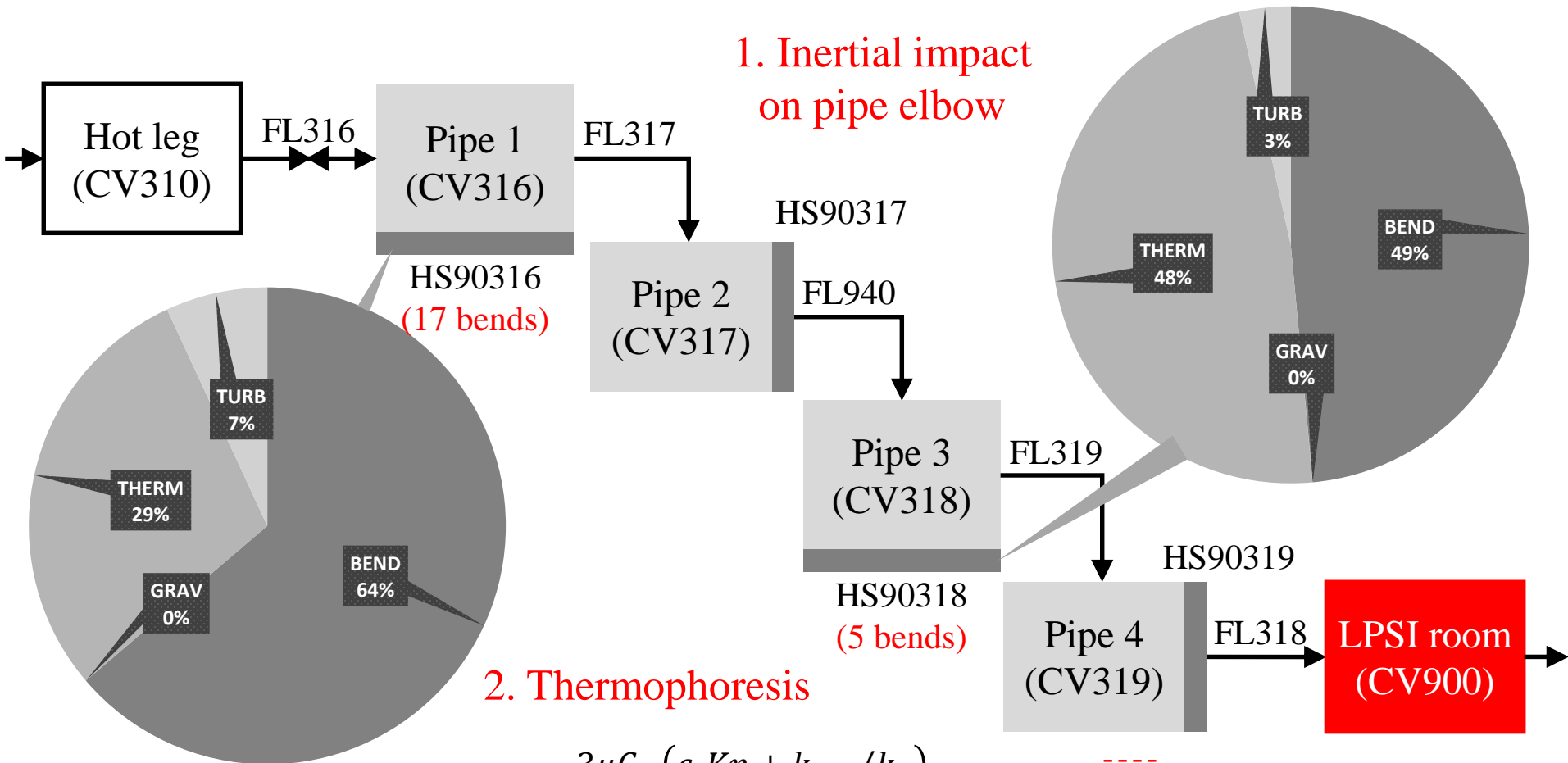
$$DF_{CSI} = DF_1 \times DF_{2,d} \times DF_{3,d} \times DF_{4,d}$$

$$DF_{I_2} = DF_1 \times DF_4 \text{ in early accident}$$

→ 3 Iodine chemistry(CsOH)

CsI deposited on SCS pipeline

Shutdown Cooling System(~89 m l, ~0.25 m d), pipe, elbow, tee, nozzle, valve, ...



$$v_{therm} = \frac{3\mu C_m (c_t Kn + k_{gas}/k_p)}{2\chi\rho_{gas}T(1 + 3F_{slip}Kn)(1 + 2c_t Kn + k_{gas}/k_p)} \quad \nabla T > 100 \text{ K}$$

CsI released into pool formed in AB

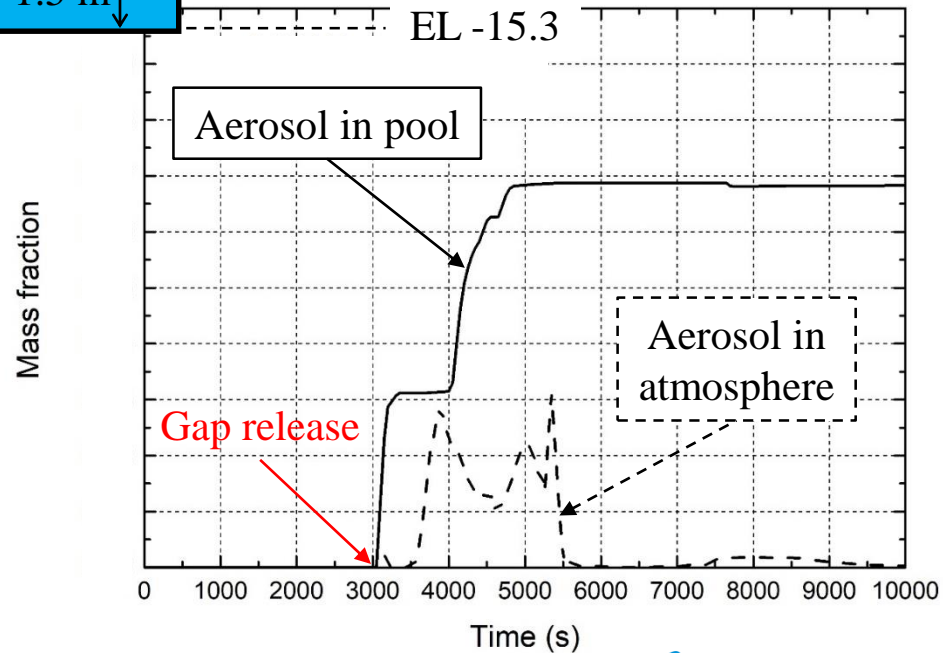
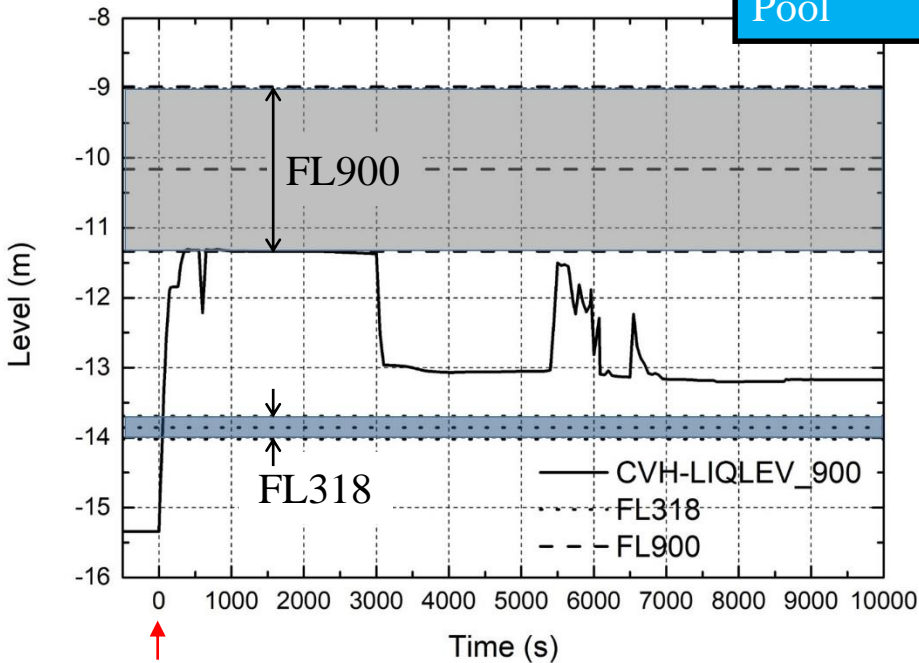
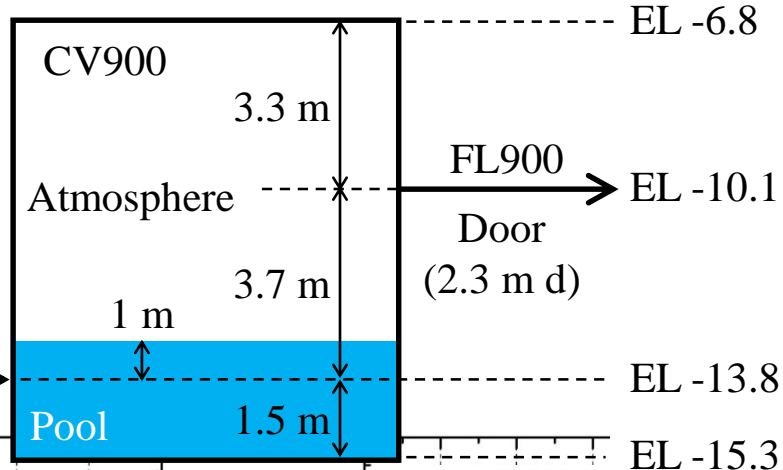
LPSI room(371 m³)

P: 101 kPa

T: 368 K(pool), 519 K(atm)

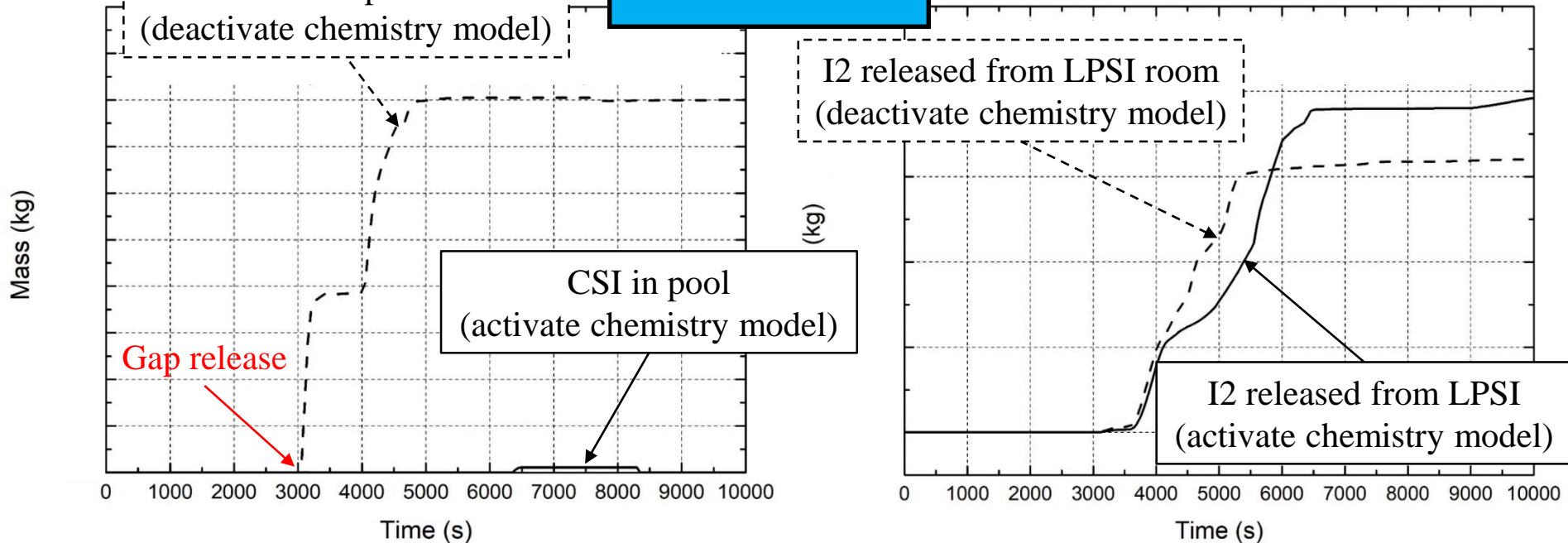
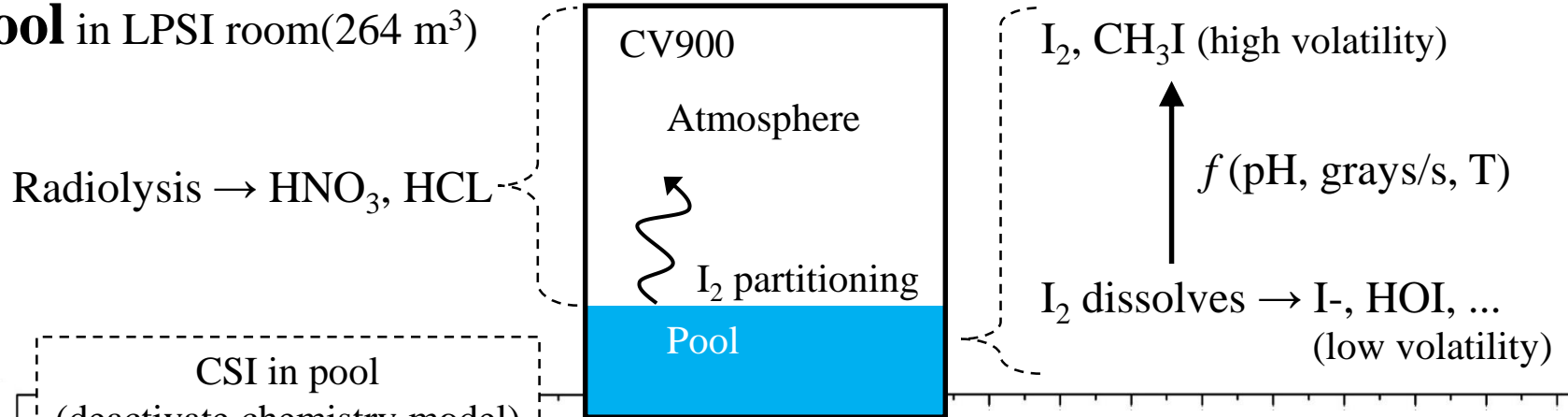
V: 137 m/s

SCS pipe(0.45 m d) → FL318



Iodine pool chemistry in AB

Pool in LPSI room (264 m³)



Conclusion and Future work

“Development of Mitigation System for Containment Bypass Accident
in Nuclear Power Plant”

- 2018(1_{st} year) MELCOR input to simulate ISLOCA in OPR1000⁽¹⁾
- 2019(2_{nd} year) Thermal hydraulic analysis(P, T, V) → Pool in AB⁽²⁾
- 2020(3_{rd} year) Behavior of fission products in SCS and AB
- 2021(4_{th} year) Sensitivity analysis of mitigation action
 - Cooling pipe, water level, spray, ...
- 2022(5_{th} year) Analysis for development of mitigation system

(1)KAERI/TR-7290/2018, (2)SAMRC 2019

This work was supported by the Korea Institute of Energy Technology Evaluation and Planning (KETEP) grant funded by the Korea government (Ministry of Trade, Industry and Energy)
(No. KETEP-20181510102400).