

alumina pellet and cladding is filled with He in the pressure range of 20-50 bar. The ramp rate is targeted to 5-10 K/s based on the typical reflooding condition used in the previous study [1,2]. The ramp rate and internal pressure could be changed to serve a test purpose. The typical experimental conditions are summarized as shown in **Table I**.

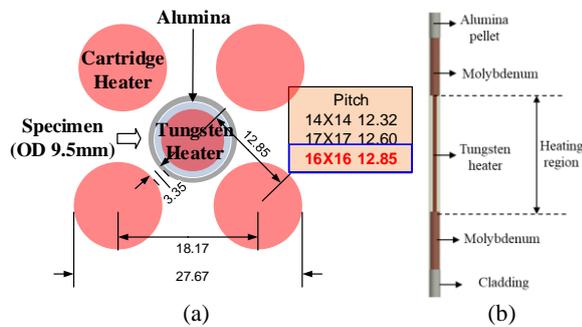


Fig. 2. Configuration of assembly of test section
(a) cross-section of NEST-1, 2, (b) inner-heater

Table I: Experimental conditions

	Description
Environment	Inert (Ar) : NEST-1,2 Steam : NEST-2
Rod array	cruciform shape (surrounded by heated guard rods)
Ramp rate	5-10 K/s
Internal pressure	20-50 bar (He)

3. Performance tests

To confirm the functions of experimental setup with target ramp rate (5-10 K/s), the performance tests of NEST-1 and NEST-2 were performed, respectively. **Fig. 3** shows the photo of NEST-1 and its configuration for performance test. In this test, only inner heated only by inner heater (ramp rate ~ 3.8 K/s) and 50 bar of inner pressure were maintained.

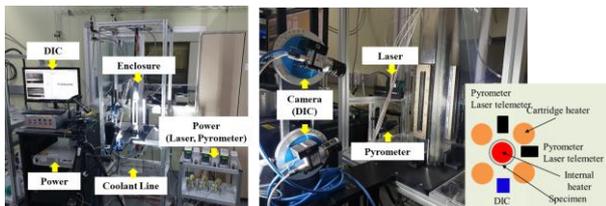


Fig. 3. Photo of NEST-1 and the configuration for performance test

As shown in **Fig. 4**, temperature ramped from 450 °C to 785 °C and cooled after burst. During the test, all of data were successfully measured including multi-dimensional / in-situ deformation data using DIC and

laser telemeter. The performance test of NEST-2 was also successfully performed. As shown in **Fig. 5**, the large deformation and burst phenomenon of cladding were observed, respectively.

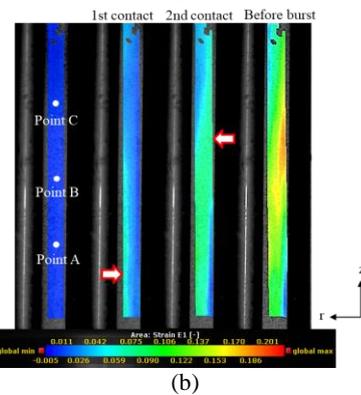
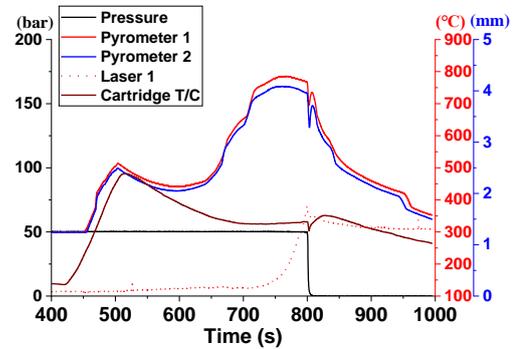


Fig. 4. Measured data of NEST-1 performance test
(a) temperature, pressure and deformation by laser telemeter,
(b) deformation by DIC

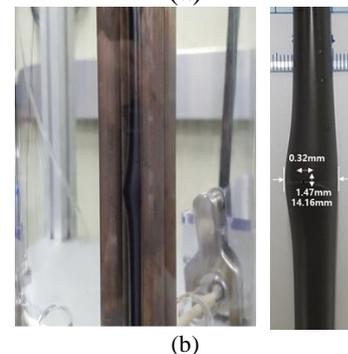
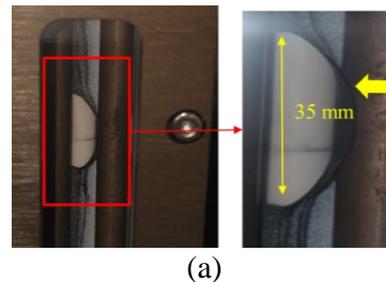


Fig. 5. Burst specimen during performance test
(a) NEST-1, (b) NEST-2

To apply the test results into validation data of multi-rod analysis model, temperature distribution of specimen and cartridge heater were measured during experiments. K-type T/Cs with the sheath of 0.5mm diameter were spot-welded on the surface of specimen and cartridge heaters as shown in **Fig. 6**. The multiple tests were performed with various combination of power and gas flow. The temperature distribution of specimen and cartridge heaters in one of temperature distribution test of NEST-2 are shown in **Fig. 7**. The target ramp rate (5-10 K/s) was accomplished with 7.6 K/s and the maximum cladding temperature reached to 818.9 °C. The maximum differences in the axial 80 mm area were 34 K on a cladding and 37 K on cartridge heaters, respectively. The maximum differences in the azimuthal direction 10 K on a cladding and 20 K on cartridge heaters, respectively.

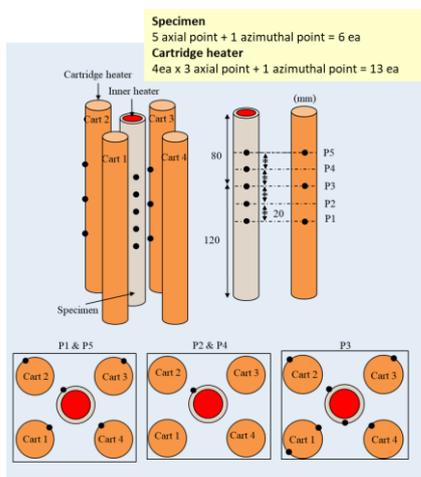


Fig. 6. T/C welding locations for temperature distribution test of NEST-2

The optimization of ramp rate and the temperature difference between cartridge heater and specimen would be controlled by additional tests.

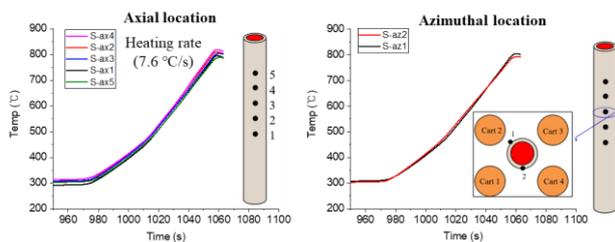


Fig. 7. Temperature distribution of the specimen in NEST-2

4. Conclusion

Neighboring rods Effect Simulation Tester (NEST), which are experimental setups for large deformation of cladding considering neighboring rods, has been developed to validate multi-rod analysis model. NEST-1 was designed to acquire real-time data on multi-

dimensional deformation in inert environment, and NEST-2 was designed to validate the deformation and oxidation models in steam environment. The performance tests were performed to confirm the functions of experimental setup with target ramp rate and the temperature distributions for further code analysis. According to the results of performance tests, the test matrix would be determined. From the further tests, the studies are planned to investigate the differences in the ballooning/burst behavior in multi-rods with that in single rod such as long ballooning and resulting the increase in flow blockage area.

ACKNOWLEDGEMENT

This work has been carried out under the Nuclear R&D Program supported by the Ministry of Science and ICT. (NRF-2017M2A8A4015024)

REFERENCES

- [1] NEA, Nuclear fuel behaviour in loss-of-coolant accident (LOCA) conditions:- State-of-the-art Report. 2009, Nuclear Energy Agency
- [2] C. Grandjean, A state-of-the-art review of past programs devoted to fuel behaviour under LOCA conditions Part One. Clad swelling and rupture assembly flow blockage, in Technical Report SEMCA-2005-313. 2005, IRSN
- [3] A.K. Yadav, C.H. Shin, S.U. Lee, H.C. Kim, Experimental and numerical investigation on thermo-mechanical behavior of fuel rod under simulated LOCA conditions, Nucl. Eng. Des. 337 (2018) 51-65
- [4] D.-H. Kim, G.-H. Choi, H. Kim, C. Lee, S.U. Lee, J.-D. Hong, H.-S. Kim, Measurement of Zircaloy-4 cladding tube deformation using a three-dimensional digital image correlation system with internal transient heating and pressurization, Nucl. Eng. Des. 337 (2020) 110662