

‘2000

가

Turbine Output Evaluation according to Operation at Reduced Reactor Coolant Temperature

103-16

1,2 , 1

1 가 (PWSSC) 가

Palo Verde 1,2,3 3,4

1

가

Control valve (Valve Wide Open)

, 1 10 9.3% 3.5%

Abstract

Steam generators usually have several kinds of damages such as primary and secondary side of tube stress corrosion cracking, fretting wear etc. Among them, Primary Water Stress Correction Cracking (PWSSC) are related with material properties, residual stress coming from manufacture and operating temperature. But only operating temperature can be controlled without hardware change. Actually, Palo verde 1,2,3 nuclear station which is model plant of Yonggwang nuclear unit 3,4 are operating at reduced RCS temperature for the reduction of the primary side corrosion cracking. The life of steam generator tube can be elongated by ORT (Operation at reduced temperature), while turbine output comes to reduction. This paper evaluated steam pressure variation from ORT and detail turbine output due to reduced steam

temperature. This evaluation is based on turbine control's valve wide open(VWO) condition for power recovery of turbine output. As a result, main steam pressure at 10 reduced RCS temperature reduced to 9.3% of rated power, and turbine power reduced to about 3.5% of rated turbine output(1049.461MWe).

1.

3,4

621.2 , 564.5 , 60.75 × 10⁶lb/hr
 450 , 2250psia , 1070psia
 6.36 × 10⁶lb/hr (moisture separator) steam drier가
 0.25% 가 8214 8%

가 (1)

2.

$$Q_p = W_p C_p (T_h - T_c) \quad (1)$$

$$Q_s = W_s (h_s - h_{fw}) \quad (2)$$

, T_h T_c , W_p W_s C_p h_s h_{fw} 가
 (steady state)

$$W_p C_p (T_h - T_c) = W_s (h_s - h_{fw}) = UA \Delta T_{lm} \quad (3)$$

, U

가 100%

가

가

2

(1)

$$T_c = T_h - \left(\frac{\dot{Q}}{\dot{V}} \right)_{ref} \cdot \frac{v_p}{C_p} \quad (4)$$

v_p (T_c) 2250 psia C_p

$\frac{Q}{V}_{ref}$ 3,4 (T_h=621.2 , T_c=564.5)

3,4

$$W_p = \dot{V}_{ref} \cdot v_p \quad (5)$$

SAFE

SAFE

ABB-CE
(2)

가

(1)

8%

가

가

가

가 10.2

611

9.3%

11.1% 가

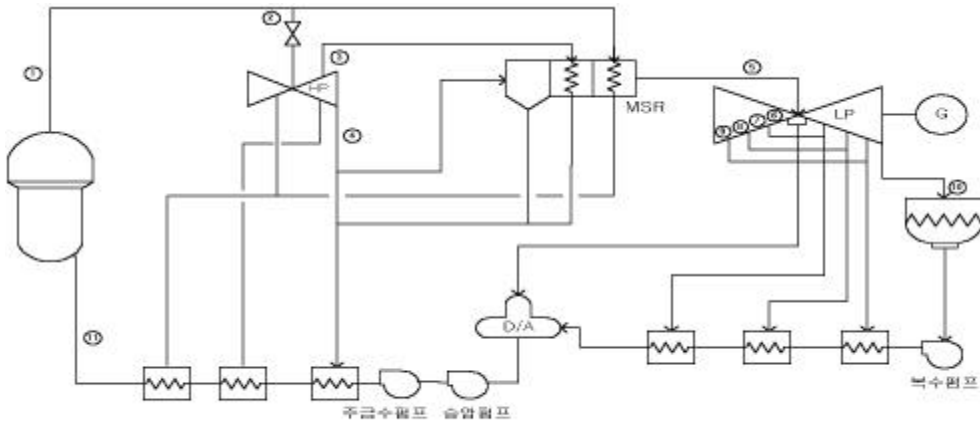
(1)

T_h ()	T_c ()	P_s (psia)	T_s ()	m_s ($\times 10^6$ lb _m /hr)	V_s ($\times 10^6$ ft ³ /hr)	
621.2	564.5	1070.0	552.8	6.363	2.624	
619	562.06	1047.2	550.2	6.356	2.685	2.3%
616	558.76	1017.6	546.7	6.346	2.743	4.5%
614	556.56	998.3	545.5	6.340	2.820	7.5%
611	553.29	970.0	540.9	6.332	2.914	11.1%
608	550.03	941.2	537.3	6.324	3.028	15.3%

3.

(1) 3,4

(3)



(2)

	ORT	5	7	10
(Mw)	2825(100%)			
(psia)	1070(1092.8)	1039.3	1019.6	990.7
(%)	0.9975	0.9975	0.9975	0.9975
(psia)	1035	1005	986.2	958.3

Plugging 8%
Plugging 0%

가

3.1

THB

Control

VWO
1.5 in Hg, 0.9,

가

(TTD) 2.8 , (DCATD) 5.6
3.27%

3.2

(2)

가 4.5%, 7.5%, 11.1%, 15.3% 가 (3)

VWO
100%

1049.461MWe

4.9%, 6.7%, 9.3%, 12.3%

5, 7, 10, 13

1.2%

가, 0.7% , 3.5%

(4) ORT

가

가

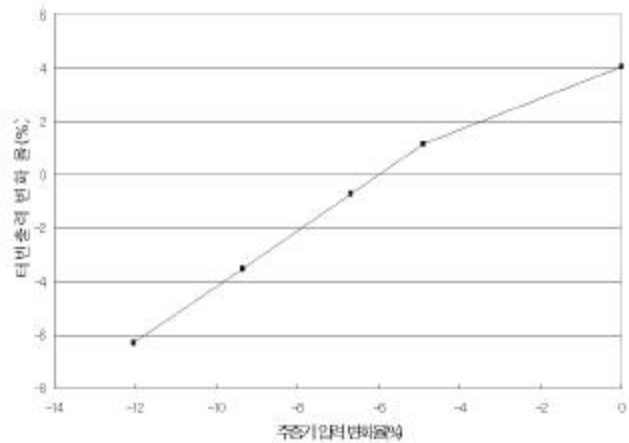
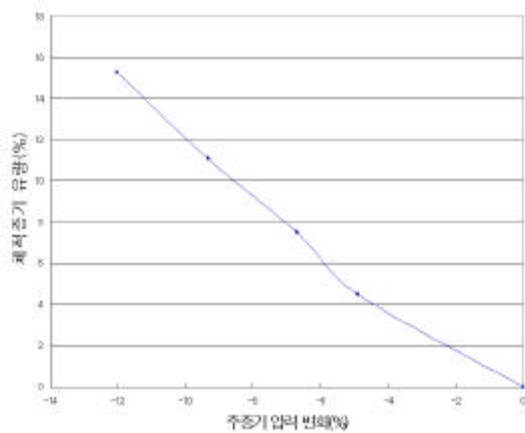
100%

(6) ORT

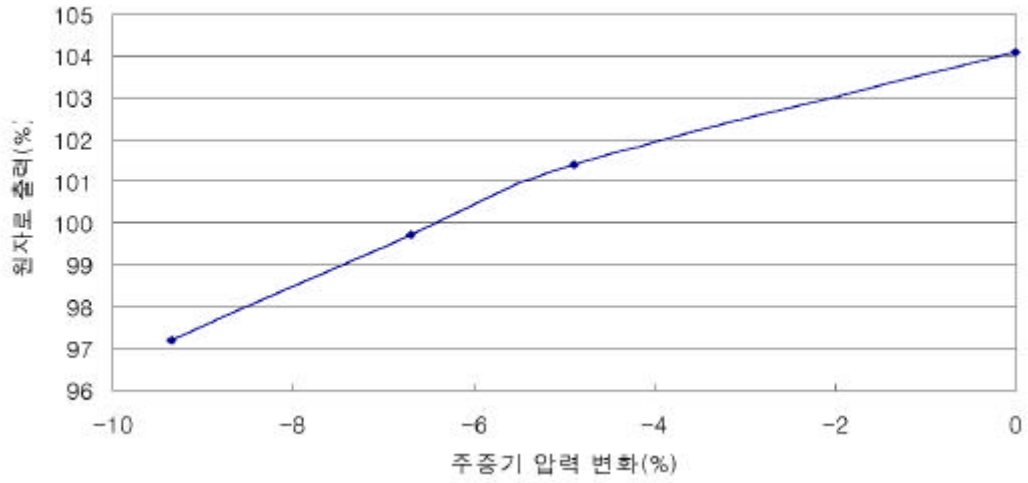
가

(2)

(3)



(그림 4) 주증기 압력변화에 따른 원자로 출력변화(%)(VWO)



4. 가

4.1

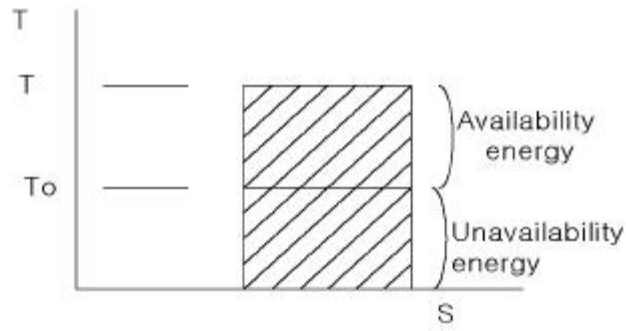
가 가 가
 가 5% (2) (4)
 10 ORT 1 97.2%

2 가 (4,5,6)
 (surroundings) (To), (Po)
 가

(Potential energy) 가,
 (zero)
 (Availability) 가 가 가

, 2
 $S = Q_0 / T_0$, $W_{max} = Q - T_0 S$
 Q 가 가 가 가

(5) Carnot T-S



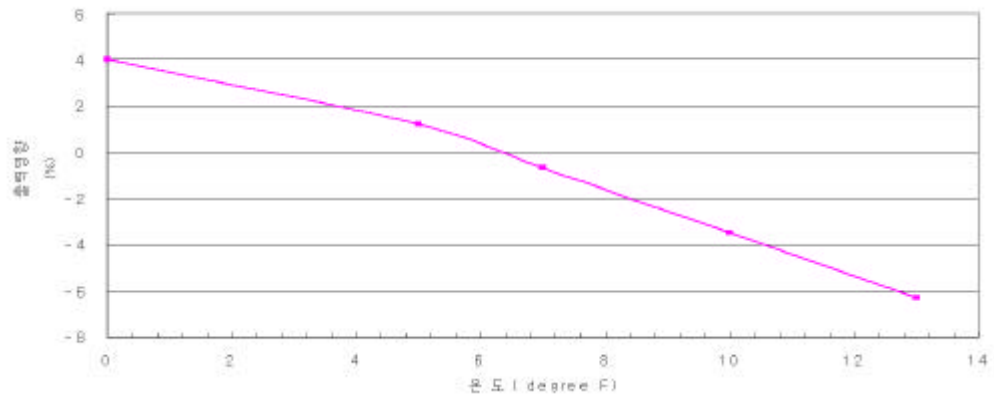
4.2 ORT

(6)

6 7

가 ,
 100%
 6 7 가
 가 ,

(그림 6) ORT 온도 범위별 터빈출력 변화 (VVO 조건)



4.2

4.2.1

ANO-2 1,2 4% , San
Onofre 2,3 1,2,3
100% 가 .

4.2.2
1 가 가
가 가
10 ORT 가
55% By-pass 100% 가
27 2.2% 가
3.5% 1.3%

5.
3,4
5, 7,
10, 13 4.9%, 6.7%, 9.3%, 12.3% , VWO
1.2% 가, 0.7% , 3.5%
가 가 가
98.7%

- 1) 3&4 FSAR 5.4.2
- 2) U- , TM.95ZJ16.P1998.785
- 3) 2 “ 3,4 ” 1996.8
- 4) Richard E. Sonntag , Gordon J.Van Wylen "Introduction to thermodynamics " 3rd 1990
- 5) J.B.Jones and G.A. Hawkins, "Engineering Thermodynamics", John Willey & Sons, 1960
- 6) K.H.Choi, "Thermal performance analysis of Yonggwang unit 1 according to reactor coolant temperature change", 1996

(7) 3,4 (9.3% , VWO)

