

Effects of carbon and nitrogen on the properties of austenitic stainless steel for nuclear power plants

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Na 가
 ton Na
 4mm
 가가
 가 가
 가

Abstract

LMFBR uses Na as a cooling material not water. Na has a property to react with water and air promptly and its corrosivity is very high. Then the thickness of steam generator tubing should be designed more than 4 mm. Like these, the properties of steam generator tubing are very important for the safety of LMFBR. Therefore, the systematic and long-term research about the materials for LMFBR should be proceeded. To simulate the selective removal of alloying elements in LMFBR, we made some stainless steels varying carbon and nitrogen contents and did evaluate the microstructure, mechanical properties, and corrosion resistance.

1.

Na Monju, 1 2
 1,700 ton Na 220 ton
 Na 가
 ton Na
 4mm
 가 melt-down
 1995 12 8
 Monju Na-leak, 10
 가 Na-leak
 1 2 Na-loop
 , 1965 가
 가 가
 , Na-Na Na-
 Prototype LMFBR IHX
 Na/
 LMFBR
 Na-
 EBR-II Na- 가 2
 (1). 15
 가 2.25Cr-1Mo 575-711K 9,000 가
 가 23.3 4.5% , 가
 CRBRP (2).
 Phenix 2.25Cr-1Mo , 2.25Cr-1Mo-1Nb (), 321
 () 3 가
 (3). (>1,000MWe) /
 Super Phenix 800

BN-350 LMFBR(2.25Cr-1Mo)

PFR(2.25Cr-1Mo/ 316)

가
가

가 (4). 가 가

800 가

800 가

Na

Na

S32050

가

가

2.

2.1

S32050 2 3

soaking 4 mm 가 1250°C 2

2.5 mm

2.2

1150 5 가

. TT(Thermal Treatment) 가 550

15 , 75 , 150 , 300 , 500 Ar

2.3

Alumina 0.05 μ m
 , 10ml HNO₃ + 90ml H₂O 5V 30

Table 1. Chemical composition of the experimental alloys(wt%)

Alloys	S32050L	S32050H	S32050(std)	SRN 1	SRN 2	SRN 3	Modified 9Cr-1Mo
C	0.01	0.08	0.02	0.027	0.026	0.027	0.096
Mn	0.84	0.76	0.92	0.87	0.84	0.82	0.44
Si	0.53	0.51	0.20	0.50	0.44	0.49	0.06
Fe	bal.	bal.	bal.	bal.	bal.	bal.	bal.
P	0.006	0.006	0.020	0.030	0.031	0.028	<0.02
S	0.003	0.003	0.002	0.002	0.002	0.002	<0.01
Cr	23.2	23.3	22.2	20.7	20.8	21.4	8.95
Mo	6.2	6.26	6.1	6.1	6.1	6.0	0.48
N	0.31	0.38	0.26	0.01	0.18	0.35	0.045
Ni	21.2	21.2	21.6	23.3	22.8	23.0	0.19

2.4

SiC 600
 7
 Mitutoyo ARK-600 Rockwell 5, 100kgf
 B-Scale (520°C) Cross head
 speed 5 mm/min, 520°C 30

2.5

가 (CERT, Constant Extension Rate Test) Cross Head Speed 4.41x10⁻⁶cm/sec
 40% NaOH, -900mV(SCE) 가
 가 35mm 가

2.6

TT

TT (550°C)

HNO₃ () 65% HNO₃ () 24 25%

2.7

1cm²

SiC 600 1cm²
Potentiostat(Model 173, EG&G)

10

, 10

150mV

1 mV/ sec

50°C

0.5N HCl + 1N NaCl, 50°C 1.5N HCl, 40% NaOH , 40% NaOH

3.

3.1

:

Na

가

2

9Cr-1Mo

2

9Cr-1Mo

가

가

9Cr-1Mo

9Cr-1Mo

2-3

9Cr-1Mo

9Cr-1Mo

가

Table 2-1. Tensile strength(MPa) of the experimental alloys at room temp. and 520 .

Test temp. \ Alloys	SR50AL-A	SR50AL-TT	SR50AH-A	SR50AH-TT	9Cr-1Mo
Room temp.	745.2	769.1	799.4	826.4	549.0
High temp.(520)	563.4	559.1	623.3	613.8	421.9

Table 2-2. Yield strength(MPa) of the experimental alloys at room temp. and 520 .

Test temp. \ Alloys	SR50AL-A	SR50AL-TT	SR50AH-A	SR50AH-TT	9Cr-1Mo
Room temp.	356.5	378.6	391.5	421.9	403.9
High temp.(520)	228.8	219.7	249.0	236.2	321.9

Table 2-3. Elongation(%) of the experimental alloys at room temp. and 520 .

Test temp. \ Alloys	SR50AL-A	SR50AL-TT	SR50AH-A	SR50AH-TT	9Cr-1Mo
Room temp.	48.3	55.9	44.5	55.6	21.0
High temp.(520)	61.2	62.2	58.2	59.8	26.3

Table 2-4. Area breakdown(J) of the experimental alloys at room temp. and 520 .

Test temp. \ Alloys	SR50AL-A	SR50AL-TT	SR50AH-A	SR50AH-TT	9Cr-1Mo
Room temp.	102.2	142.2	123.6	162.3	45.1
High temp.(520)	102.3	105.1	124.1	116.2	39.3

: 3 25% 24
 가 65% 600
 690 Modified Huey Test
 , 9Cr-1Mo

Table 3. Corrosion rate of the C-controlled super stainless steels by modified Huey test(boiling 25% HNO₃, 24 hrs immersion)

Alloys	Corrosion Rate	
	mpy	mdd
SR50AL(A)	0.84	4.67
SR50AL(TT)	0.50	2.77
SR50AH(A)	0.42	2.31
SR50AH(TT)	0.26	1.47
9Cr-1Mo steel	1586.3	8592.6

4 40% NaOH -900mV(SCE) 가
 . 9Cr-1Mo

Table 4. SCC results obtained from CERT in boiling 40% NaOH

Alloys	9Cr-1Mo	SR50AL (annealed)	SR50AL (TT)	SR50AH (annealed)	SR50AH (TT)
Elongation, %	16.1	26.2	38.4	34.9	40.2

3.2

: 1
 가
 2 가

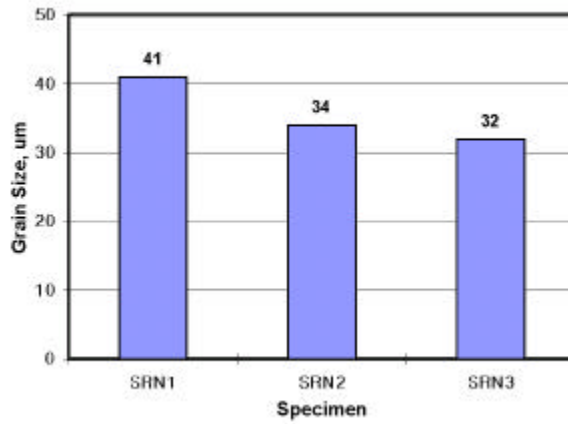


Fig. 1. Effect of N on the grain size of annealed super stainless steels

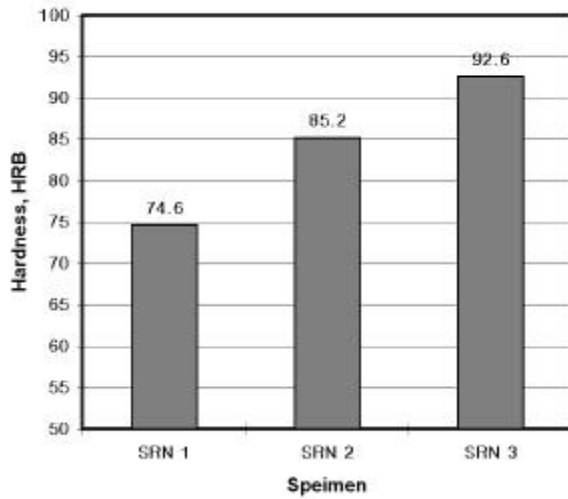


Fig. 2. Effect of N on the hardness of annealed super stainless steels

3
 가 가 가
 4 550°C 500 (TT)
 가 가
 TT 가 5
 가 가 가 (0.01%N) TT TT
 가 가 가 6
 0.18% SRN2 TT
 가 TT 가

가 TT 가 가 가 TT 가

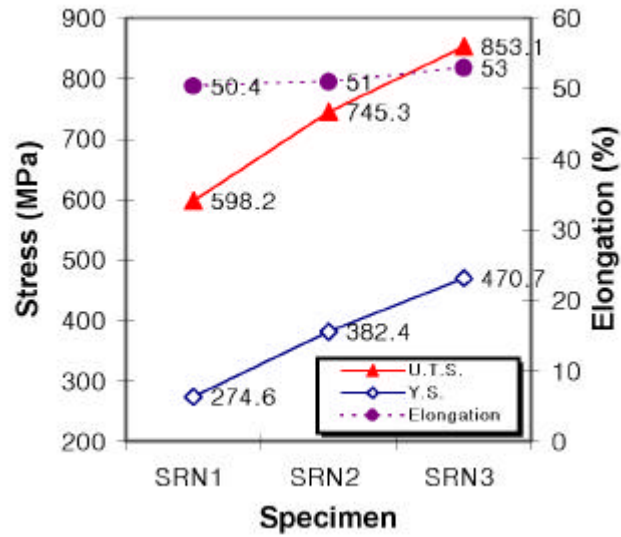


Fig. 3. Effect of N on the tensile properties(room temp.) of annealed super stainless steels

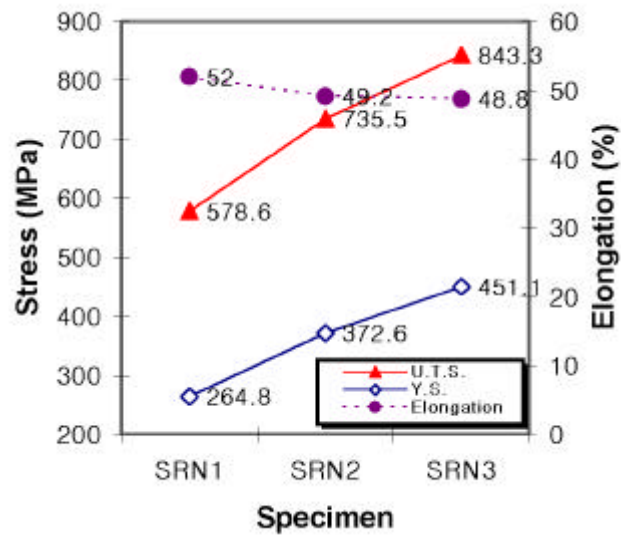


Fig. 4. Effect of N on the tensile properties(room temp.) of thermally treated super stainless steels

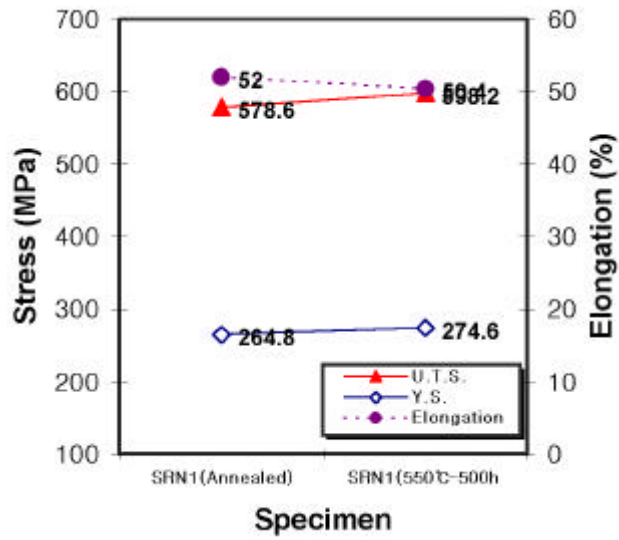


Fig. 5. Effect of thermal treatment on the tensile properties(room temp.) of N-free stainless steel - SRN1

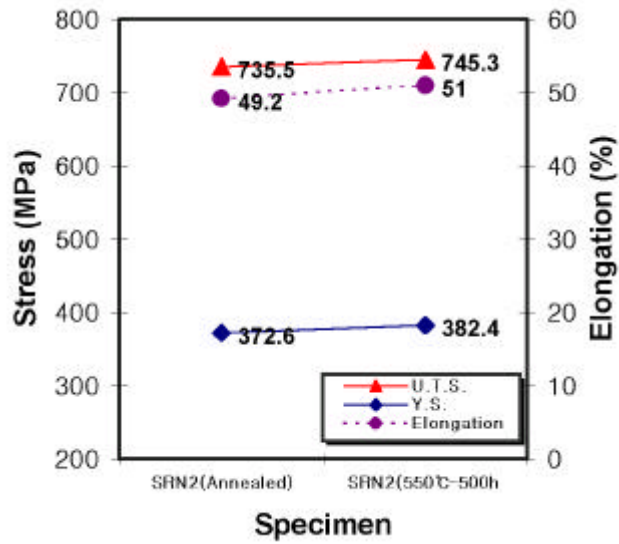


Fig. 6. Effect of thermal treatment on the tensile properties(room temp.) of N-bearing stainless steel - SRN2

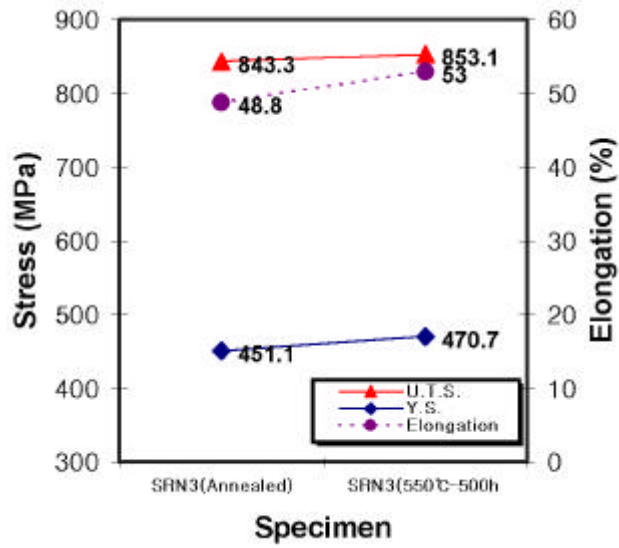


Fig. 7. Effect of thermal treatment on the tensile properties(room temp.) of high N-bearing stainless steel - SRN3

	8	50°C, 0.5N HCl + 1N NaCl	1mV/ sec		
		가		가	
		가		가	Cr Mo
		가		가	
9		50°C, 1.5N HCl	1mV/ sec		
		가		가	Cr Mo
가				가	
				가	
10		40% NaOH	1mV/ sec		
		가		가	가
가				가	가
		SRN1		가	
가				가	
				SRN1	가
		가			가
11		65%		24	

가 가 25%
 가 가 65% 가 가
 10 mpy 가 가
 가 가 TT 가 가
 TT 가 가 TT
 가 가 가

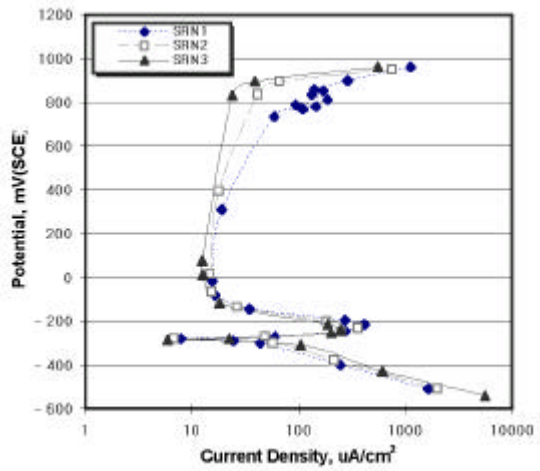


Fig. 8. Effect of N on the anodic polarization behavior of annealed specimen in deaerated 50°C 0.5N HCl + 1N NaCl at 1mV/ sec

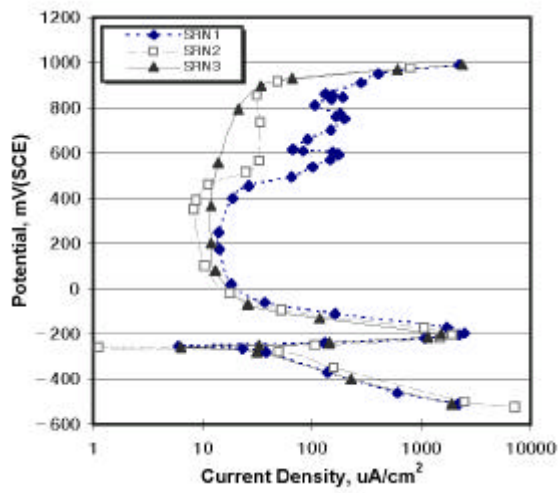


Fig. 9. Effect of N on the anodic polarization behavior of annealed specimen in deaerated 50°C 1.5N HCl at 1mV/ sec

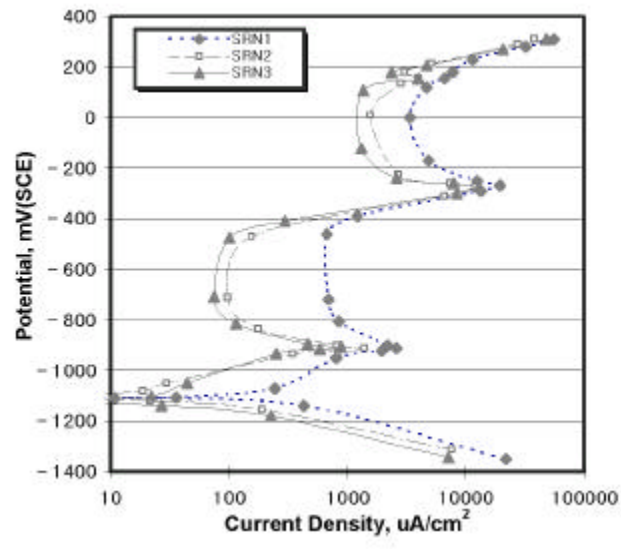


Fig. 10. Effect of N on the anodic polarization behavior of annealed specimen in boiling 40% NaOH at 1 mV/ sec

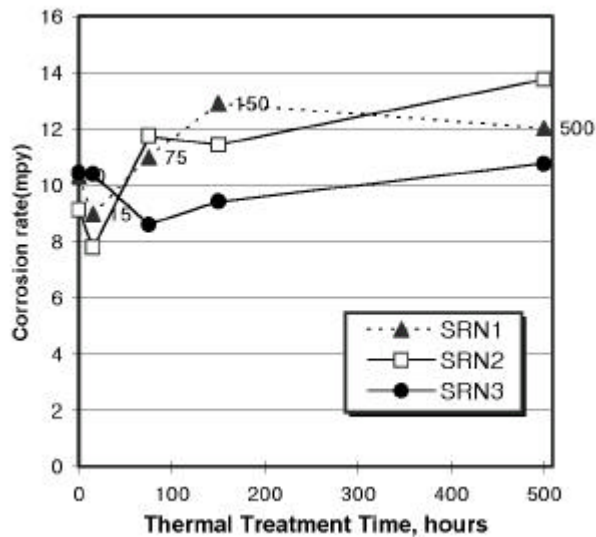


Fig. 11. Effect of thermal treatment(550°C) on corrosion rate in boiling 65% HNO₃

3.

3.1

- (1) 9Cr-1Mo 가 가 .
- (2) 9Cr-1Mo 가 가 .
가 TT ,
- (3) TT 가 가 가 가 .
- (4) Modified Huey Test , 9Cr-1Mo S32050
, S32050 .
- (5) 40% NaOH , 9Cr-1Mo S32050
가 TT .

3.2

- (1) 가 , 가 .
TT 가
가 TT 가 .
- (2) TT 가 ,
TT 가 .
- (3) 가 가
가 가 가 가 .
- (4) 65% 가 TT

가 가 , 가 TT
가 가 .
가 .

1. J.A. Shields, Jr. and K.L. Longua, "The effect of ten years experimental breeder reactor II service on 2.25Cr-1Mo steel", Nuclear Technology, 28, 471-481(1976)
2. J.C. Whipple and C.N. Spalaris, "Design of the Clinch River Breeder Reactor Plant steam generators", Nuclear Technology, 28, 305-314(1976)
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