

CFD

**Application of CFD method to optimal design of swirl-vane
in a nuclear fuel assembly**

150

(Computational Fluid Dynamics)
(swirl-vane)

가 가
(40°)

가 가
가

가
가

Abstract

This study applied the CFD method to the optimal design of the swirl-vane that is invented to enhance the thermal-hydraulic performance of a nuclear fuel assembly. The main parameters of the swirl-vane configuration are the slope angle of the vane supporter and the bend angle of the vane. This study conducted a numerical analysis of the heat transfer in pipe and the flow characteristics in a rod bundle with the swirl-vane in order to propose an optimal vane angle. The flow mixing and heat transfer were enhanced due to the stronger swirling flow as the vane angle increases. However, beyond a critical vane angle of 40°, the enhancement of flow mixing appears to decrease and the pressure drop significantly increases.

1.

(subchannel) (swirl)

DNB

DNB

가

Westinghouse (split-vane)
(4692302[1])

가

가

가 가

ABB-CE

(side-supported vane)

(5440599[2])

가

90°

SPC

가

Siemens

(5402457[3])

가

가

(swirl-vane)

[4]

4

가

3

가

Karoutas

[5] split-vane

3

CFD

CFDS-FLOW3D

CFD

Imaizumi [6]

3

CFD

[7]-

[10] CFD

CFX[11]

가

CFD

(1)

CFD

1

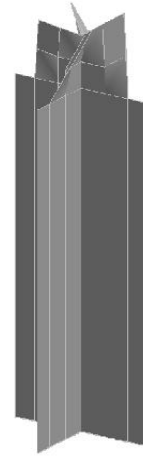
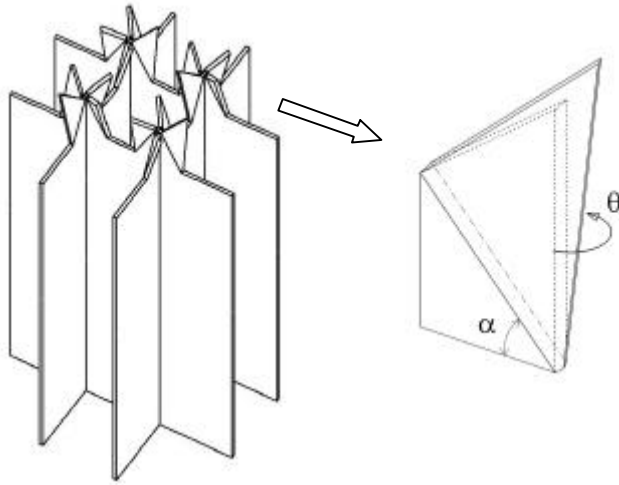
(a)

(q)

20°, 30°, 35°, 40°, 45°

50°

CFD



CFD

CFD

1.

2. CFD

2.1

가 (CHF) 가 1
 CHF (R134)
 CFD CFX[11] (single phase)
 CHF 가 3 (structured grid) 4
 1 (4) CFD
 mm (d) 12 mm (35 mm) 30
 , 500 mm 2
 32
 250
 CFD 가
 no slip
 가 ()
 (G) 2000 kg/m²s (Re=24000) 288K 가 ()
 300 Kw/m² 가 가 25°, 30°, 35°

40°

가

Launder Spalding[12]

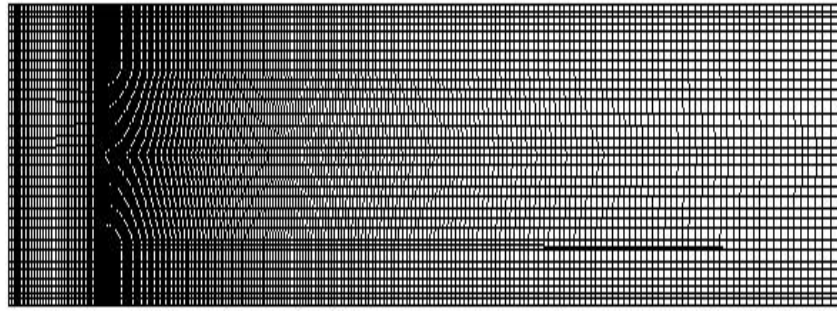
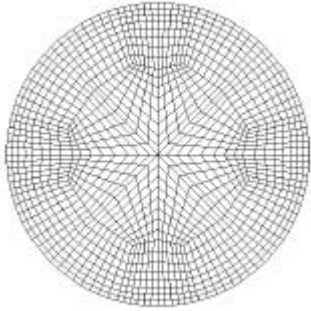
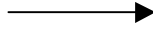
$k-e$ Hybrid
(residual) $10^{-3} - 10^{-4}$ 가

4000-5000

HP9000 C200

(PA8000 CPU, 512 MB RAM)

z



2.

(32x32x250)

2.2

CFD

(D) 9.5 mm (P)

1.33D

(D_h) 12 mm

(35mm)

600 mm

30 mm

CFD

(,)

CFD

3 CFD

250000

3

(gap)

36 ,

16

(z) ()

245

CFD

가

()

gap

gap

gap

가

no slip

$V_{bulk}=6.8$ m/s (Re=80000)

Launder

Spalding[12]

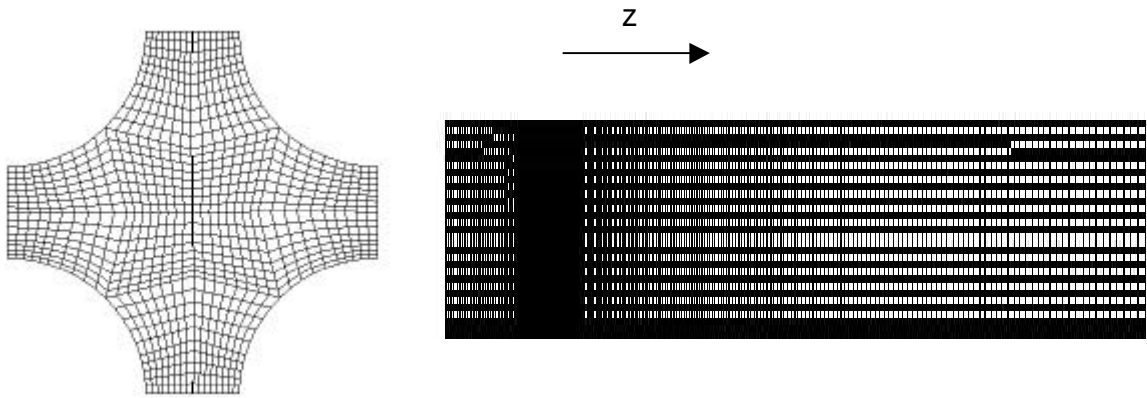
$k-e$ Hybrid

(residual) $10^{-3} - 10^{-4}$ 가

5000

가 25°, 30°, 35°, 40°, 45°

가



3.

(36x36x245)

3.

(swirl)

4

(q)가 30°

가

(gap)

가

가

[10, 13]

S_M

$$S_M = \frac{\int_0^R r^2 V_{lateral} U dr}{R \int_0^R r U^2 dr} \quad (1)$$

R

(P/2)

U

$V_{lateral}$

()

5

(q)

(1)

가 0°

(0)가

$$S_M = S_0 \exp(-bz/d)$$

가

35°

가

가 가

가

$q = 40^\circ$

$q = 35^\circ$

($z < 10d$)

가

$q = 35^\circ$

가

(b)

$q = 35^\circ$

0.025

($q = 25^\circ, 30^\circ, 40^\circ$)

$b = 0.03$

17%

5

($q = 30^\circ$)

(subchannel)

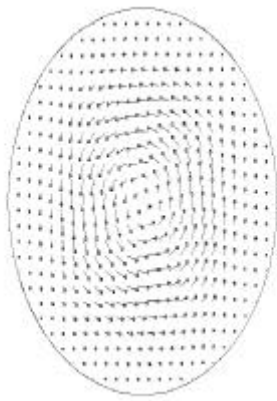
(gap)

가

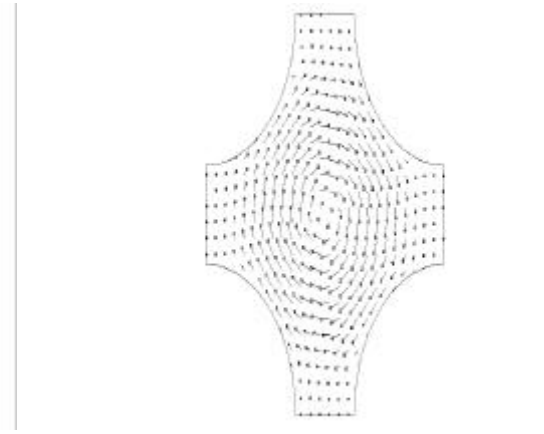
($b = 0.06$)

가

가

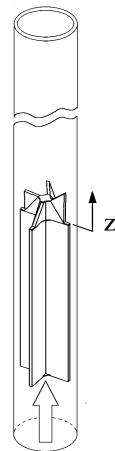
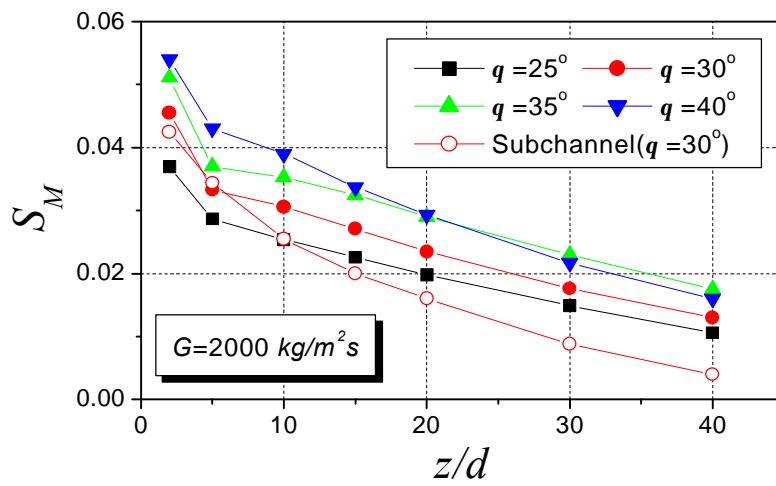


4 ($z=5d$)



($z=5D_h$)

($q = 30^\circ$)



5.

6

가

($z=5d$)

가

가

가 가

가

가

가

가

가

가

가

$q = 35^\circ$

가 가

$q = 25^\circ$

가

$$S_{M, total} \equiv \int S_M d(z/d) \quad (2)$$

$$\Delta T_{W, max} = \frac{T_{W, max}(q) - T_{W, max}(0)}{\Delta T_W(0)} \quad (3)$$

(3) $\Delta T_W(0)$

가

가

4.16K

7

(q)

($S_{M, total}$)

가

($\Delta T_{W, max}$)

가 가

가

$q = 35^\circ$

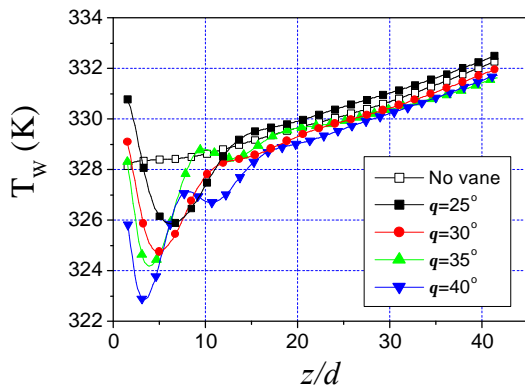
가 가

$q = 35^\circ$

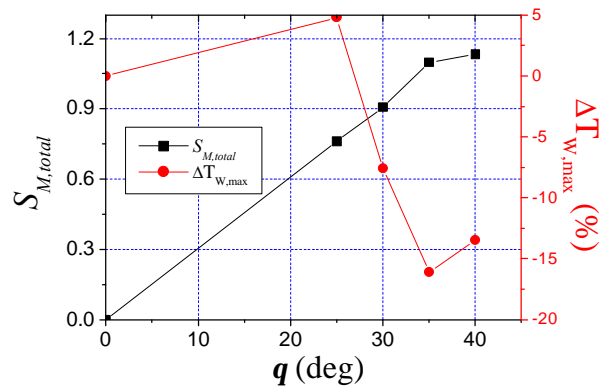
가

가

35°



6.



7.

가

8

()

gap

가

($z=2.1D_h$)

(V_{bulk}) 10%

$z=8.5D_h$

10%

(q)가 가

()

가

가

$q = 45^\circ$

$z=8.5D_h$

$q = 40^\circ$

9 (1)

(S_M)

가 가

($z < 10D_h$)

가

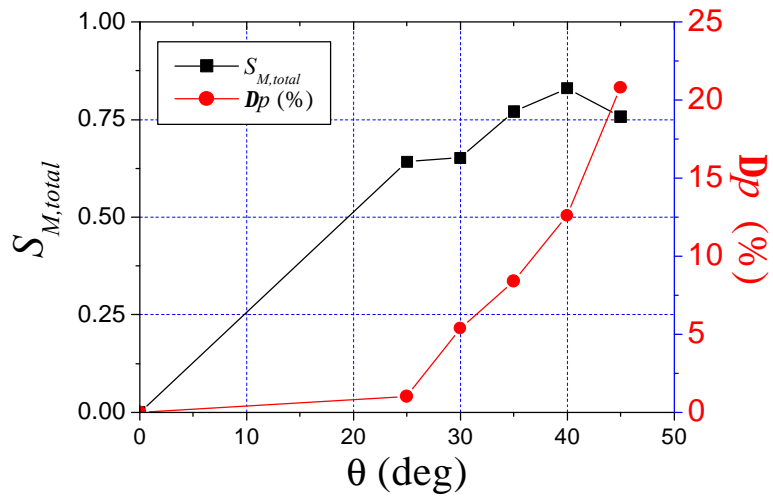
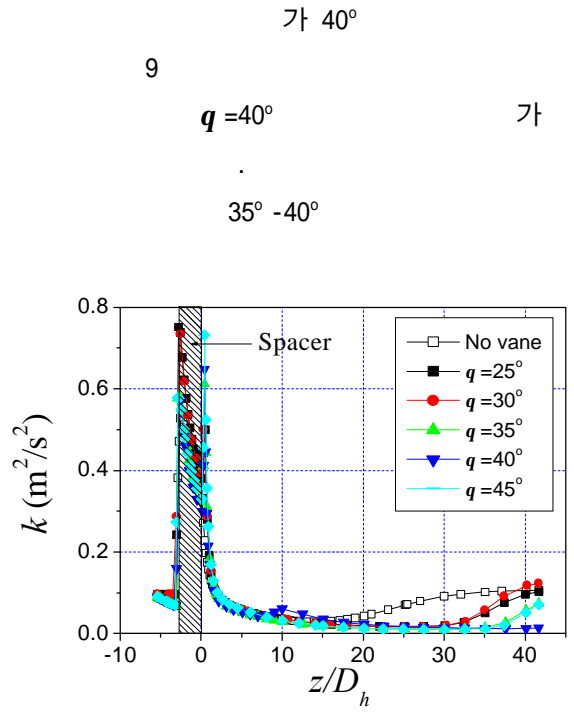
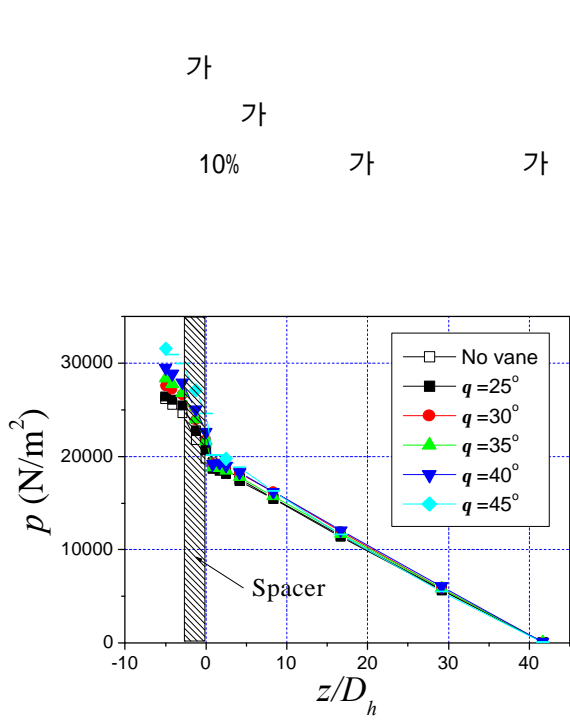
$$(S_M = S_0 \exp(-bz/d))$$

b

0.05 ($q = 25^\circ, 35^\circ, 40^\circ$)

0.06 ($q = 30^\circ, 45^\circ$)

($b = 0.03$)



4.

(q)가 가 가 가 가 q = 35° 가 가 가 가 q = 35° 가 가 가 가 가 가 가 가 가 가

$q = 40^\circ$

가

가

$35^\circ - 40^\circ$

- [1] Edmund E. et. al., "Coolant Flow Mixer Grid for a Nuclear Reactor Fuel Assembly," United States Patent 4692302, Westinghouse, September 8, 1987.
- [2] Thomas Rodack et al., "Spacer Grid with Integral Side Supported Flow Directing Vanes," United States Patent 5440599, Combustion Engineering, August 8, 1995.
- [3] Peter Suchy et al., "Fuel Assembly with a Grid Structure between the Rods," United States Patent 5402457, Siemens, March 28, 1995.
- [4] , , 가 , , 98-3133, 1998.
- [5] Karoutas Z., Gu C.Y., and Scholin B., 1995, 3-D Flow Analysis for Design of Nuclear Fuel Spacer, Proc. of the 7th Int'l Meeting on Nuclear Reactor Thermal-Hydraulics, Sept. 10-15, New York.
- [6] Imaizumi, M., Ichioka, T., Hoshi, M., Teshima, H., Kobayashi, H. and Yokoyama, T., 1995, "Development of CFD method to evaluate 3-D flow characteristics for PWR fuel assembly,"
- [7] , , , , 1998, " CFD , " '98 , .
- [8] W.K. In, D.S. Oh, T.H. Chun and Y.H. Jung, 1998, "CFD application to turbulent flows in rod bundles with mixing blades," NTHAS98: First Korea-Japan Symposium on Nuclear Thermal Hydraulics and Safety, Pusan, Korea, 1998.
- [9] W.K. In, D.S. Oh, T.H. Chun and Y.H. Jung, 1999, "CFD analysis of turbulent flows in rod bundles for nuclear fuel spacer design," Trans. of the 15th International Conf. on Structural Mechanics in Reactor Technology (SMIRT-15), Seoul, Korea, August.
- [10] , 1999, " , " '99 , .
- [11] AEA Technology, CFX-4.2 Solver, Harwell Laboratory, Oxfordshire, UK, 1997.
- [12] Launder B.E. and Spalding D.B., 1974, "The numerical computation of turbulent flows," Computational Methods in Applied Mechanics and Engineering, 3, 269-289.
- [13] , , , 1999, " , "

