

TRANSX/DANTSYS

LWR MOX

Application of TRANSX/DANTSYS to LWR MOX Benchmark Calculations

150

TRANSX/DANTSYS  
 PNL30~35 . ENDF/B-VI 가 NJOY  
 MATXS-format 69 .  
 ,  $k_{eff}$   
 Pitch 가  $k_{eff}$  가  
 Pitch  $k_{eff}$   
 MCNP .

Abstract

The benchmark calculations with TRANSX/DANTSYS code system was performed against PNL30~35 critical experiments to verify the code system's applicability to LWR MOX core. The MATXS library of 69 neutron groups for light-water-reactor problem was generated using NJOY code through processing the recent release of ENDF/B-VI nuclear data. Relative power distributions except outermost lattice fuel rods showed reasonable results, and the calculated  $k_{eff}$  of all cases showed good consistency with benchmark-model values or MCNP results. Moreover, calculated results indicate that there is an increase in  $k_{eff}$  with increasing lattice pitch, and showed the tendency that  $k_{eff}$  value for the borated case was higher than that for the unborated case with the same pitch.

1.

가 (Evaluated Nuclear Data File)

Los Alamos

NJOY 가

, 가 ENDF-6 format  
 (Monte Carlo) MCNP  
 Reference 가 , (Discrete Ordinates)  
 DANTSYS 가 , 가 .  
 1 ANISN 2 DOT-IV  
 Oak Ridge  
 3 가 DOORS .  
 DOORS 3.1 , 1 ANISN 2 DORT, 3 TORT가  
 , ANISN DORT 가  
 , DANTSYS Los Alamos 1, 2 3  
 , NJOY  
 MATXS-format TRANSX  
 DANTSYS , 1 ONEDANT  
 2 3 TRANSX/DANTSYS .  
 LWR (MOX) 가 ENDF/B-VI  
 MATXS-format TRANSX/DANTSYS LWR  
 MOX .

## 2.

MATXS-format 가 ENDF/B-VI Release 5<sup>[1]</sup>  
 , NJOY <sup>[2]</sup> 97.95 94.105 가 . NJOY  
 RECONR, BROADR, HEATR, UNRESR, THERMR, GROUPE, MATXS  
 , NJOY 97  
 MATXS 가 ,  
 MATXS 94.105 .  
 Source .  
 가 LWR  
 , WIMS-D 69  
 ENDF/B-VI 가 Pointwise Tolerance 0.2%

## 3. TRANSX/DANTSYS

### 3.1 TRANSX

TRANSX <sup>[3]</sup> MATXS-format (Diffusion) , , 가 가 , Adjoint , , , 가 가 , MATXS /가 . 2.15 , ASCII CARD, FIDO, ANISN Binary ISOTXS (Material-Ordered) GOXS(Group-Ordered) . , GOXS-format Material-Ordered Group-Ordered GIP . TRANSX Bondarenko , Dancoff 가 가 .

### 3.2 DANTSYS

DANTSYS<sup>[4]</sup> Boltzmann . Input , Solver , Edit , 1, 2, 3 ONEDANT, TWODANT, THREEDANT Solver , 2 Mesh Mesh TWODANT/GQ, TWOHEX Solver . Solver Iteration 가 Diffusion Synthetic Acceleration Scheme . CCCC (Committee on Computer Code Coordination) ISOTXS ANISN, FIDO-format , Quadrature Set .

## 4.

Pacific Northwest Laboratory Plutonium Recycle Critical Facility (PRCF) 1975~6 <sup>[5]</sup> MOX 6가 , PNL30~35 . International Criticality Safety Benchmark Evaluation Project (ICSBEP) ICSBEP Handbook<sup>[6]</sup> . MOX Natural UO<sub>2</sub>-2wt.%PuO<sub>2</sub>(8% <sup>240</sup>Pu) , 1.77800cm, 2.20914cm, 2.51447cm Pitch . 1.2827cm MOX 0.0762cm Zircaloy-2 Gap 1.4351cm . 92.964cm 0.5cm MOX 가 UO<sub>2</sub> . (Pure Water) , 1 PNL30~35 .

5.

5.1

NJOY MATXS-format  
 PNL30~35 MOX TRANSX/DANTSYS

TRANSX ONEDANT TRANSX TWODANT

TRANSX NJOY MATXS  
 LWR , Equivalent Cell  
 Cell

ONEDANT MOX ONEDANT  
 TRANSX , Cell  
 가

TWODANT TRANSX  
 , MOX Equivalent Cell

ISOTXS-format  
 ISOTXS TWODANT , (k<sub>eff</sub>)

TRANSX Bell-Hansen-Sandmeier ,  
 Dancoff Saurer 69  
 Legendre P<sub>3</sub>, Quadrature  
 S<sub>8</sub> TWODANT Pitch 9 16  
 Mesh , 30cm

Inner Outer Iteration 10<sup>-5</sup> , TWODANT  
 Edit Mesh 가 TWODANT  
 Buckling Height

5.2

2 TRANSX/TWODANT ICSBEP  
 Handbook MCNP ,  
 (Benchmark Model)  
 Pitch가 가 k<sub>eff</sub>가 가 , Pitch  
 k<sub>eff</sub>

MCNP , Hatice Akkurt<sup>[7]</sup>

ESADA  
 TWODANT  
 가 C/E

(Calculation to Experiment Ratio) 1~6 가  
 (Pin Number) ,  
 .<sup>[8]</sup> HELIOS  
 . PNL30 PNL31  
 . PNL32  
 1/4 , PNL32  
 가 가 , PNL34 . PNL33  
 가 , HELIOS MCNP  
 가 가  
 PNL35 .

6.

ENDF/B-VI Release 5 가 LWR 69  
 TRANSX/DANTSYS PNL30~35  
 .  $k_{eff}$  , Pitch 가  $k_{eff}$   
 가  $k_{eff}$  MCNP

[1] Cross Section Evaluation Working Group, "ENDF-102 Data Formats and Procedures for the Evaluated Nuclear Data File ENDF-6," BNL-NCS-44945 (1990)  
 [2] R. E. MacFarlane and D. W. Muir, "The NJOY Nuclear Data Processing System Version 91," LA-12740-M (1994)  
 [3] R. E. MacFarlane, "TRANSX 2 : A Code for Interfacing MATXS Cross-Section Libraries to Nuclear Transport Codes," LA-12312-MS (1993)  
 [4] Ray. E. Alcouffe et al., "DANTSYS : A Diffusion Accelerated Neutral Particle Transport Code System," LA-12969-M (1995)  
 [5] R. I. Smith and G. J. Konzek, "Clean Critical Experiment Benchmarks for Plutonium Recycle in LWR's," EPRI-NP-196, Volume I (1976)  
 [6] Hyung-Kook Joo, "Rectangular Arrays of Water-Moderated UO<sub>2</sub>-2Wt.%PuO<sub>2</sub>(8%<sup>240</sup>Pu) Fuel

Rods," MIX-COMP-THERM-002, ICSBEP Handbook (1997)

- [7] Hatice Akkurt and Naeem M. Abdurrahman, "Benchmark Calculations of the ESADA Single-Region Mixed-Oxide Critical Experiments," *Nucl. Technol.*, **127**, 301 (1999)
- [8] , , "PNL30-35 HELIOS ," '99  
(1999)

## 1. PNL

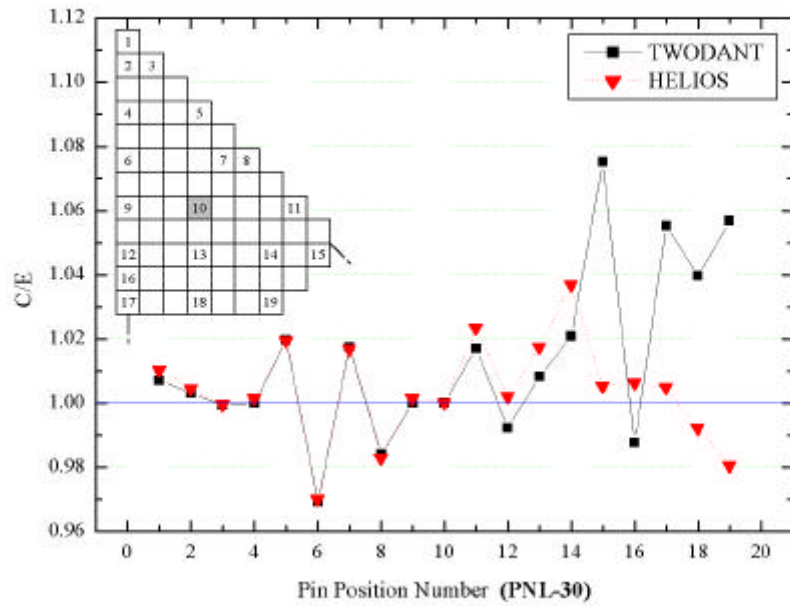
Experiment Number	Pitch (cm)	Number of Fuel Rods	Boron Concentration (ppm)	Axial Buckling Height (cm)	Temperature ( )
PNL30	1.77800	469	1.7	104.19432	20.98
PNL31	1.77800	761	687.9	102.57117	21.90
PNL32	2.20914	195	0.9	102.89525	22.75
PNL33	2.20914	761	1090.4	101.99654	22.66
PNL34	2.51447	161	1.6	100.14014	22.15
PNL35	2.51447	689	767.2	102.03419	23.40

## 2. PNL

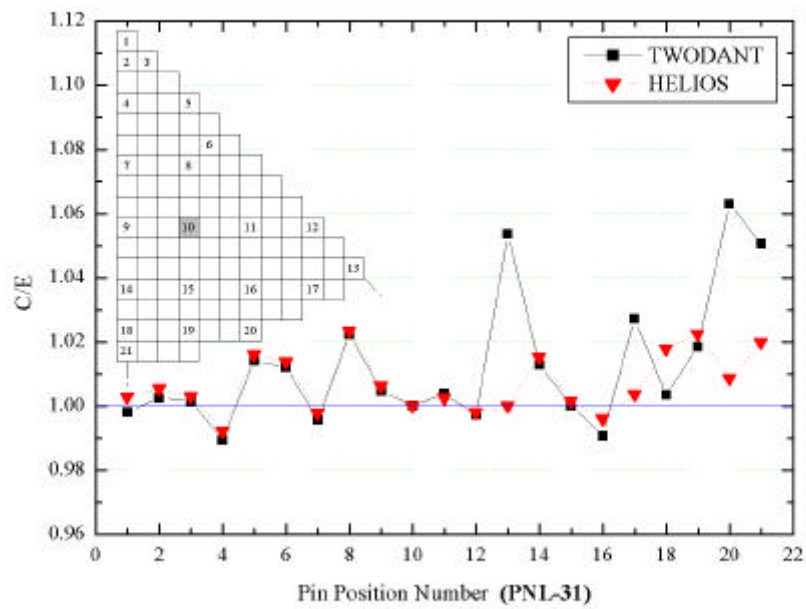
$k_{eff}$

Case	PNL30	PNL31	PNL32	PNL33	PNL34	PNL35
Experiment	1.00018	1.00006	1.00019	1.00022	1.00096	1.00013
Benchmark-Model*	1.0024	1.0009	1.0042	1.0024	1.0038	1.0029
TWODANT	0.98981	0.99321	0.99988	1.00610	1.00489	1.00713
MCNP* (ENDF/B-V)	0.9990 $\pm 0.0008$	0.9996 $\pm 0.0009$	1.0031 $\pm 0.0009$	1.0071 $\pm 0.0008$	1.0061 $\pm 0.0008$	1.0077 $\pm 0.0008$

\* Data from ICSBEP Handbook (Ref. 6)

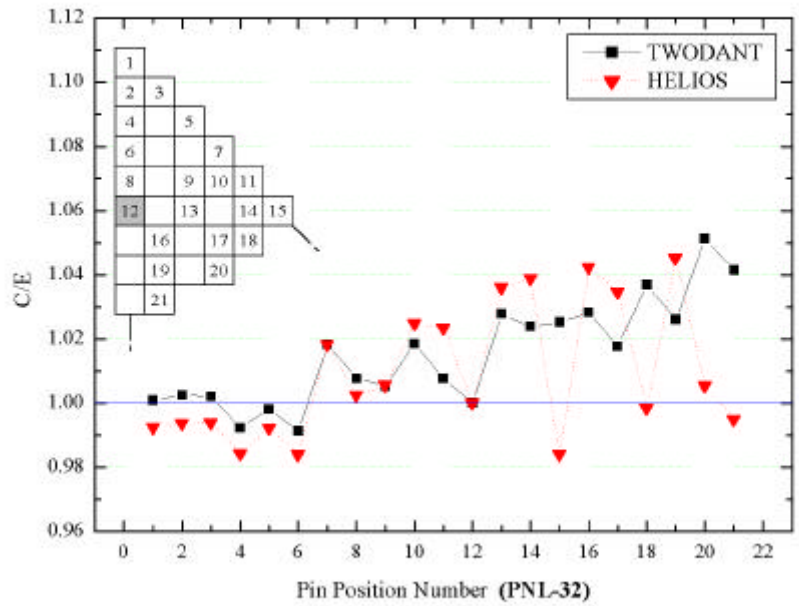


1. PNL30

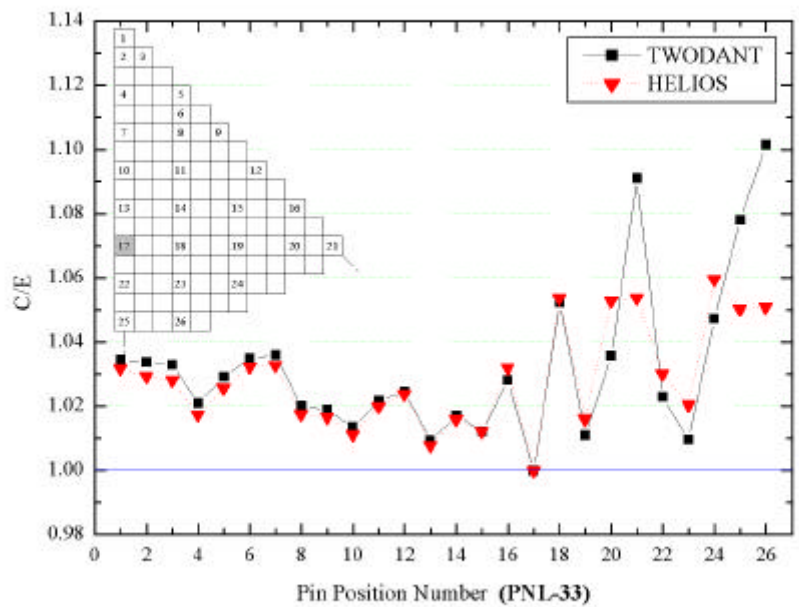


2. PNL31

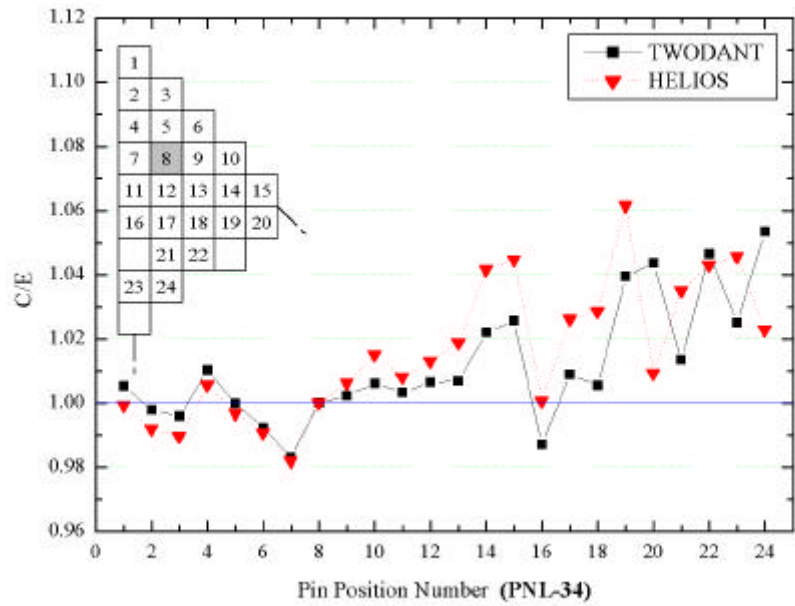




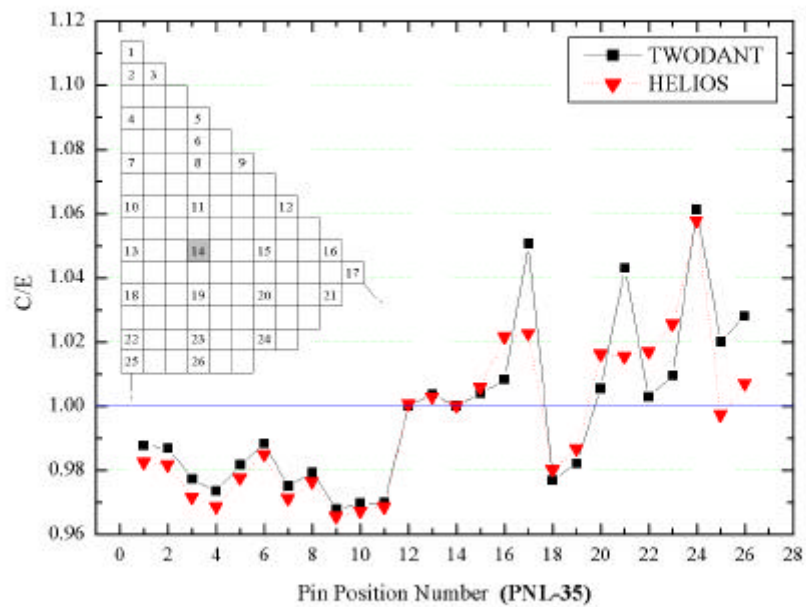
3. PNL32



4. PNL33



5. PNL34



6. PNL35