

KALIMER

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Evaluation on the Sloshing Characteristics of Reactor Vessel Internal Fluid of KALIMER Liquid Metal Reactor

220

150

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ANSYS

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**Abstract**

It is important to establish a highly accurate technique of evaluating the sloshing behavior during earthquake for structural integrity of KALIMER reactor vessel and internals. The analysis procedure of sloshing modes is established by finite element computer program ANSYS, and the effectiveness of the sloshing analysis procedure used is confirmed by comparison with theoretical and experimental results. By using this method, the relation between the design variables of reactor internal components and the characteristics of sloshing modes is analyzed and the sloshing characteristics of sodium fluid inside KALIMER reactor vessel is evaluated.

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ASCE 4-86[1]

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[2,3].

ANSYS

[4].

(UIS)

4

IHX

EMP

KALIMER

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2.

ANSYS[5]

3

(FLUID80)

. 8

FLUID80

z=0

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z=0

IHX

가

IHX

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3.

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3.2

II

(UIS)

9m

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II

2

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2

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0.58 Hz

0.52 Hz

II

4

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5

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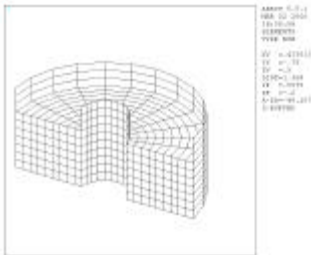
가

5

2

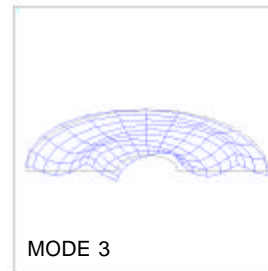
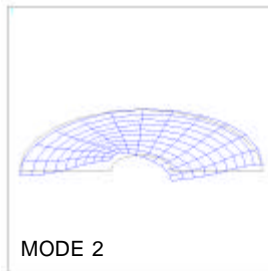
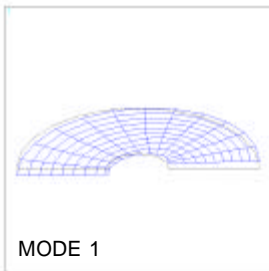
2.

II



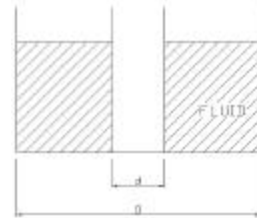
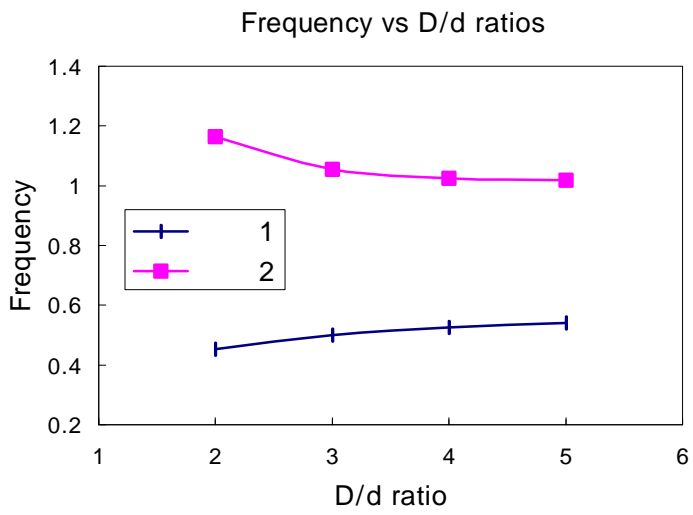
(Hz)	ANSYS	
1	0.525	0.79
2	1.024	1.00
3	1.298	0.34

3. II (D/d=4)



4.

II



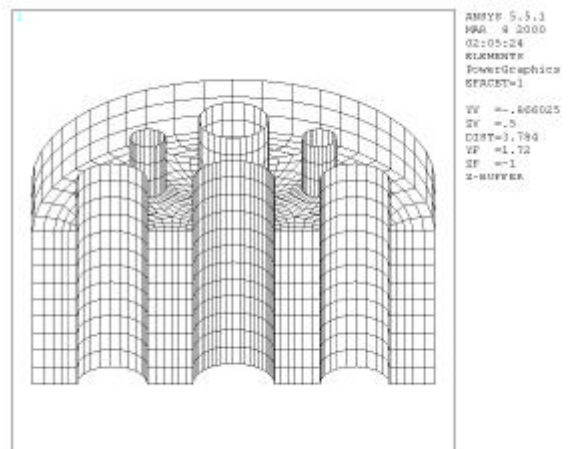
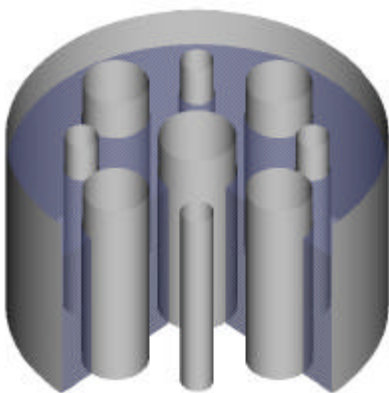
5.

### 3.3

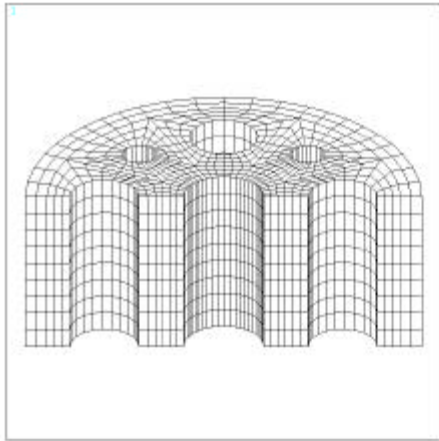
IHX EMP

(coupling)

6



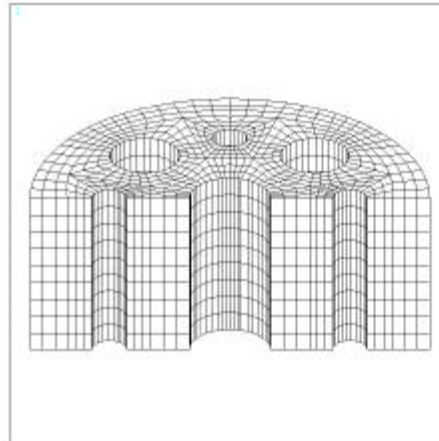
6.



```

ANSYS 5.5.1
FEM 24 2000
20:44:05
ELEMENTS
PowerGraphics
EFACET=1
TV =-.066025
TV =.5
DIST=3.784
VF =1.72
SF =-1.5
D-BUFFER
  
```

1



```

ANSYS 5.5.1
FEM 24 2000
20:47:31
ELEMENTS
PowerGraphics
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TV =-.066025
TV =.5
DIST=3.784
VF =1.72
SF =-1.5
D-BUFFER
  
```

2

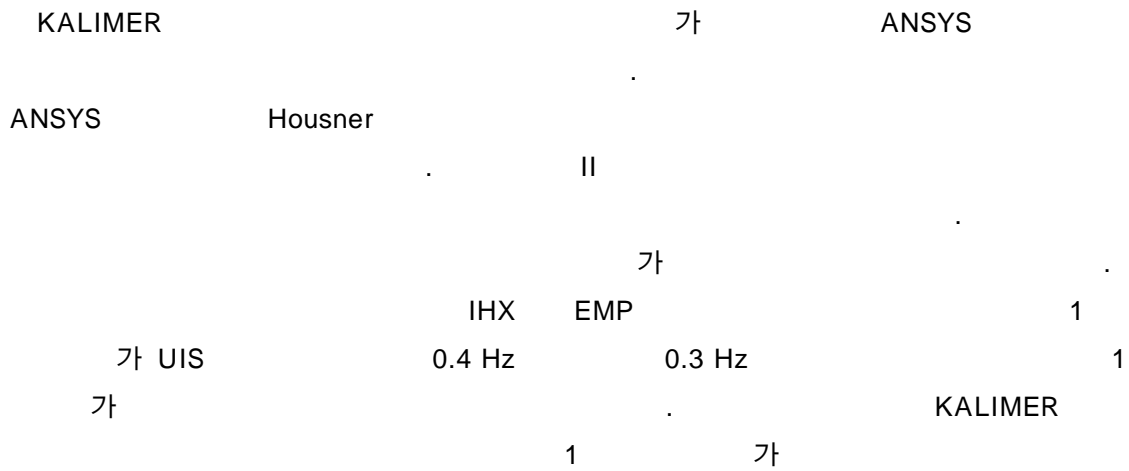
7.

IHX EMP 1 가 0.4 Hz가  
 . 3 1  
 0.3 Hz ,  
 II 2 가  
 I 1 가 3

3.

MODEL	MODE	FREQUENCY	PERIOD	PARTIC.FACTOR	RATIO	EFFECTIVE MASS
1	1	0.304730	3.2816	-75.514	1.000000	5702.42
	2	0.506135	1.9758	4.8011	0.063579	23.0506
	3	0.536354	1.8644	5.7548	0.076207	33.1172
	4	0.632131	1.5820	2.1942	0.029056	4.81433
2	1	0.306638	3.2612	-74.941	1.000000	5616.16
	2	0.502118	1.9916	3.6078	0.048142	13.0160
	3	0.545851	1.8320	-4.3568	0.058136	18.9814
	4	0.629007	1.5898	-2.5961	0.034641	6.73957

4.



1. ASCE 4-86, "Seismic Analysis of Safety-Related Nuclear Structures and Commentary on Standard for Seismic Analysis of Safety Structures", September 1986.
2. , "Fluid-Structure Interaction Analysis of Liquid Storage Structures",  
5 4 , 1992.
3. Lee, D.-G et al, "Seismic analysis of base isolated spent fuel storage structures", Transactions of the 13<sup>th</sup> International Conference on SMiRT, August 13-18, 1995.
4. , "Analysis of Sloshing and Seismic Response for Cylindrical Vessel Containing Fluid",  
, 1996.
5. ANSYS User's Manual for Revision 5.1, Swanson Analysis Systems, Inc., 1994.
6. Y. Hagiwara, Y. Masuko and Kurihara, "Development of a 3-D Vibration Analysis Code for Fluid-Structure Coupled Systems and Seismic Sloshing Analyses of Pool-Type FBR", CRIEPI, Report No. U90009, 1990.

