

Oxidation Rate Equation of Zircaloy-4 in High Temperature Steam at High Pressure

1

373- 1

LOCA -4 가 . -4
 . 700 900 1 150
 bar 가
 , 가 가 가
 가 40 60 %

Abstract

In the severe accident case like LOCA, Zircaloy claddings are oxidized not only in high temperature but also in high pressures. It is a concern whether the safety of high burn up fuels can be maintained during severe accident. The effects of steam pressure on zircaloy-4 oxidation, and the effect of pre-existing oxide layer on the cladding in the high temperature-high pressure oxidation of zircaloy-4 were investigated. Oxidation rate equation of zircaloy-4 in high temperature steam at high pressure was suggested based on the measured data. The experimental temperature range was 700 900 , and the pressures were between 1 and 150 bar. Partial pressure of steam turned out to be the important one rather than total gas pressure. The higher the steam pressure was applied, the thicker the oxide became. The effect of steam pressure on the oxidation of cladding with pre-existing oxide was about 40 60 % less effective than that of pickled cladding.

1.

1

NRC(Nuclear Regulatory Commission)

17 %

가 Baker-Just(BJ) [1]. BJ ,
가

[2].

가 , 700 900 , -4 ,
가

2.

700 900 500, 1000, 1500 ,
1 150 bar 가 1

가 thermocouple (1).

PC

400 , 3 ,
가
. 400 3 /sec ,
1.5 /sec ±5

Westinghouse 15 mm , 3 mm
(1). H₂O, HNO₃, HF(50:45:5)
55 thermocouple
(1).

가 가 가 가
 400 5 bar 가 400 5 bar
 가 75 bar 100 bar 가
 , thermocouple
 SEM(Scanning Electron Microscopy)

3.

2 가 (75, 150
 bar) 가
 (5 bar) Leistikow [3,5]
 70 bar 95 bar 가
 가 가
 가 가 [7],
 100bar
 SEM (3).

가 가 가 가 가 가 가 가
 1500 (4
). 가 (150, 100, 75, 50bar) Leistikow
 가 4 가 가 가
 , 750 800 가 가
 가 (5). 50 bar 100 bar
 가 50 bar 2 750 800
 가 가

[3,6]

가 20 μm 50 μm . 40 60 %
(6).

가 (20 μm , 50 μm)
가 , 가
가 가

4

[8 10]. 가
, 1000 가 , 1000
가 .[9,10]
1000 ,
, 1200 .

가 [4,11 14]. , 1200
가 , 가
, 가

[4,11 14], 600

920 , Lightstone Pemsler가

XRD ,

[14]. ,
Hart Chaklader 가 (oxygen-deficient zirconia)가
(superplasticity) [15]. 가 가

. Pilling Bedworth 1.5가 ,

가

[3,6]. Leistikow

400 800

50 μm

가

[3].

1000

1100

가

breakaway

1200

breakaway
, 1100

가 750 800 가 850
 , Leistikow -4 가
 [3,5], 750 800 가

$$\delta(t, 1, P) = K_T \cdot t^{0.333} = \delta_o$$

$$\delta(t, T, P) = \delta_o \cdot e^{\gamma(P-1)}$$

$$= \delta_o \cdot t^{\beta(P-1)}$$

$$\gamma(P-1) = \beta(P-1) \cdot \ln t$$

$$\beta = \frac{\gamma}{\ln t}$$

$$\therefore \delta(t, T, P) = K_T \cdot t^{\beta(P-1) + \alpha} \quad \text{----- (1)}$$

$$K_T = 2.6 \times 10^3 e^{\frac{17,960 \text{ (cal/mol)}}{1.98 \text{ (cal/mol-K)} \times T(K)}}$$

$$= 0.3333$$

$$R = 1.98 \text{ cal/mol-K}$$

: (μm)

t : (sec)

: T 가 (bar^{-1})

P : (bar)

700 900 1 (85
 0 900) , 800
 (500) (7).

NRC

LOCA

BJ

8

50 bar

BJ

-4

, BJ

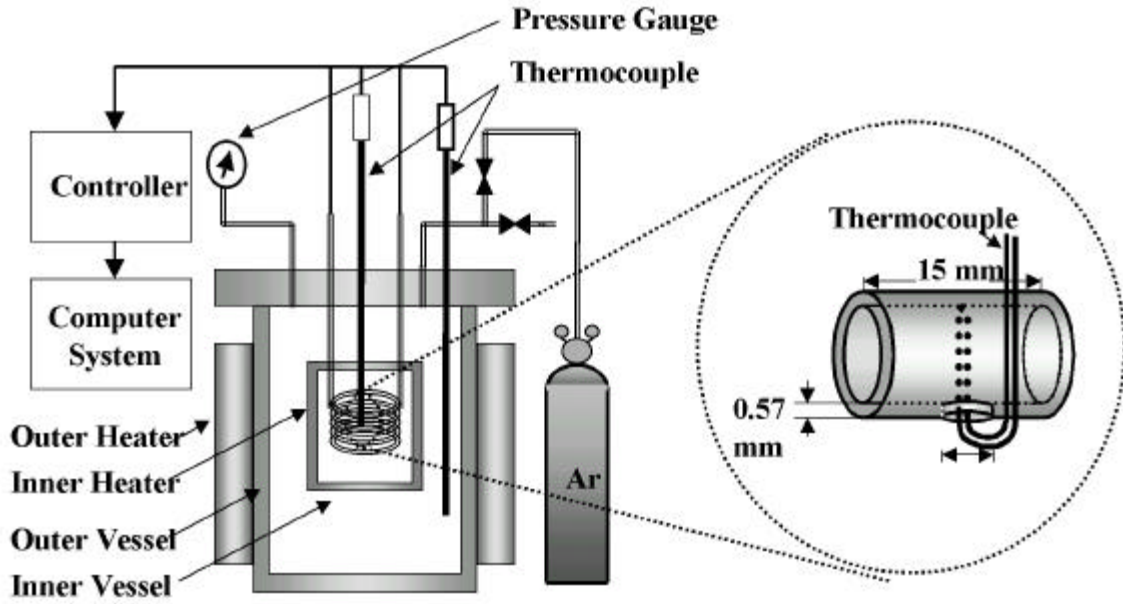
- 1)
- 2)
- 3) 1000 40 60 %
- 4)
- 5) $\delta(\mu m) = K_T \cdot t^{\beta(P-1)+\alpha}$ -4 1000 , BJ

- [1] L. Baker and L. C. Just, "Study of Metal-Water Reactions at High Temperature , Experimental and Theoretical Studies of Zirconium-Water Reaction," ANL-6548, May, 1962.
- [2] V.F. Urbanic and T.R Heidrick, J.Nucl.Mater.,75 ,251(1978)
- [3] S.Leistikow and G. Schanz, "Oxidation kinetics and Related Phenomena of Zircaloy-4 Fuel Cladding Exposed to High Temperature Steam and Hydrogen-Steam mixtures under PWR Accident Conditions," Nuclear Engineering and Design, 103 (1987) 65
- [4] C. H. Hsueh and A. G. Evans, "Oxidation Induced Stresses and Some Effects on the Behavior of Oxide Films," J. Appl. Phys., 54 (1983) 6672
- [5] S.Leistikow and G. Schanz, "The Oxidation Behavior of Zircaloy-4 in Steam between 600 and 1600 , " Werkstoffe und Korrosion, Vol. 36, pp.105-116,(1985).
- [6] S. Kass, "Corrosion of Prefilmed Zircaloy," USAEC Report, (1966) 374
- [7] R. E. Pawal, J. V. Cathcart and J. J. Campbell, "The Oxidation of Zircaloy-4 at 900 and 1100 in High Pressure Steam," Journal of Nuclear Material, 82 (1979) 129
- [8] B. Cox, "Accelerated Oxidation of Zircaloy-2 in Supercritical Steam," AECL 4448 (1973)
- [9] R. E. Pawal, J. V. Cathcart and J. J. Campbell, "The Oxidation of Zircaloy-4 at 900 and 1100 in High Pressure Steam," Journal of Nuclear Material, 82 (1979) 129
- [10] I. L. Bramwell, T. J. Haste, D. Worswick and P. D. Parsons, "An Experimental Investigation into the Oxidation of Zircaloy-4 at Elevated Pressures in the 750 to 1000 Temperature Range," ASTM STP 1245 (1994) 450
- [11] F. Gazarolli , H. Seidel , R. Tricot and J. P. Gros, "Oxide Growth Mechanism on Zirconium Alloys," ASTM STP 1132 (1991) 395
- [12] J. Pierrey, "Dilation at High Temperature," Ann. chim., 12 (1949) 133

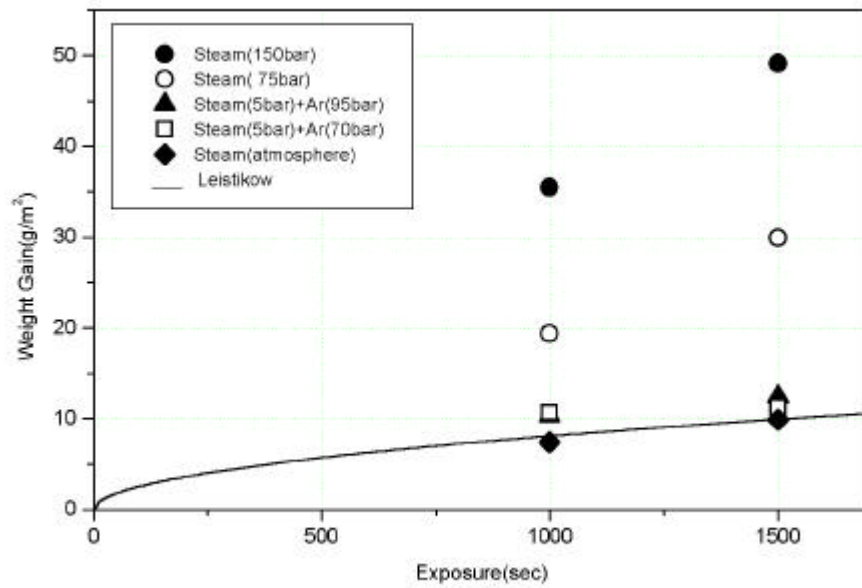
[13] J. Godlewski, J. P. Gros, M. Lambertin, J. F. Wadier, and H. Weidinger, "Raman Spectroscopy Study of the Tetragonal-to-Monoclinic Transition in Zirconium Oxide Scales and Determination on Overall Oxygen Diffusion by Nuclear Microanalysis of O¹⁸," ASTM STP 1132 (1991) 416

[14] ASTM, "Standard Test Method for Corrosion Testing of Products of Zirconium, Hafnium, and Their Alloys in 633 K or in Steam at 673 K[Metric]," Annual Books of ASTM Standards section 3, 03.02 (1991) 49

[15] C. Roy and G. David, "X-Ray Diffraction of Zirconia Films on Zirconium and Zry-3," Journal of Nuclear Material, 37 (1970) 71

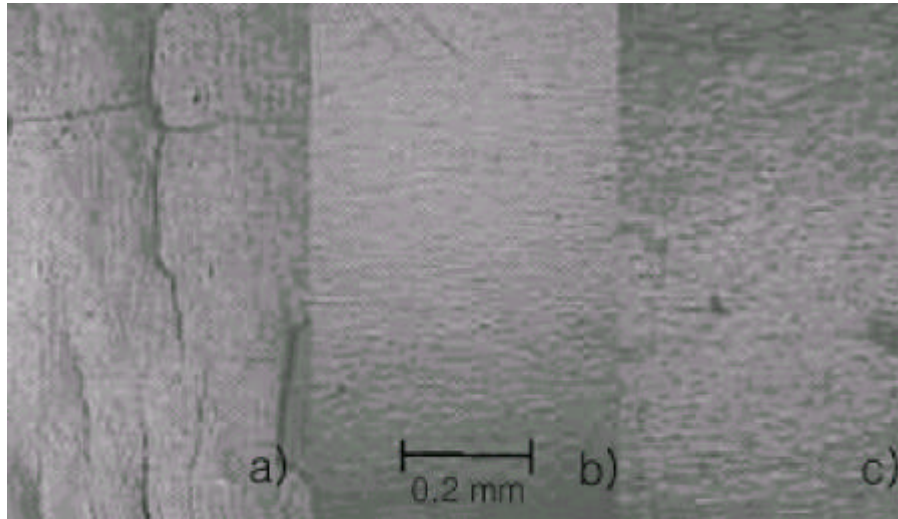


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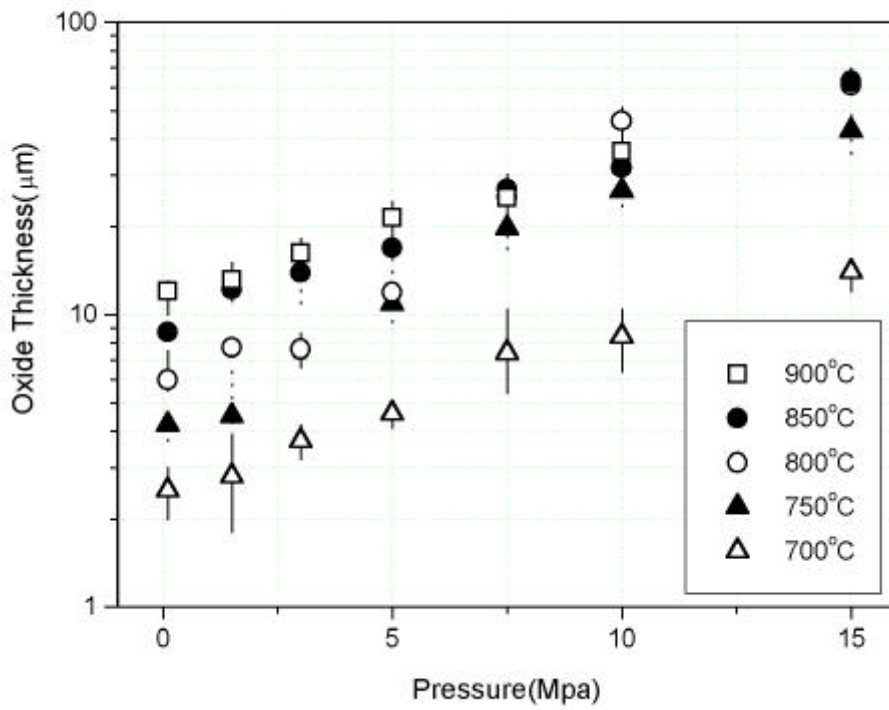


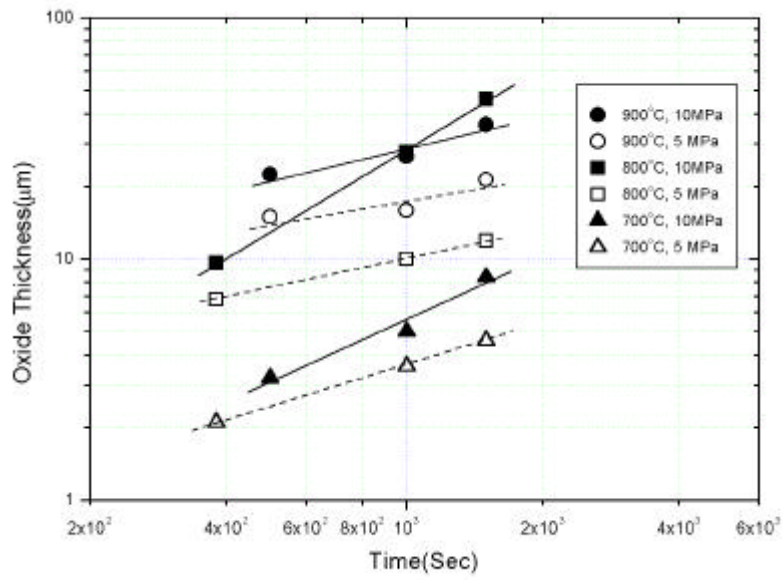
2

(750)

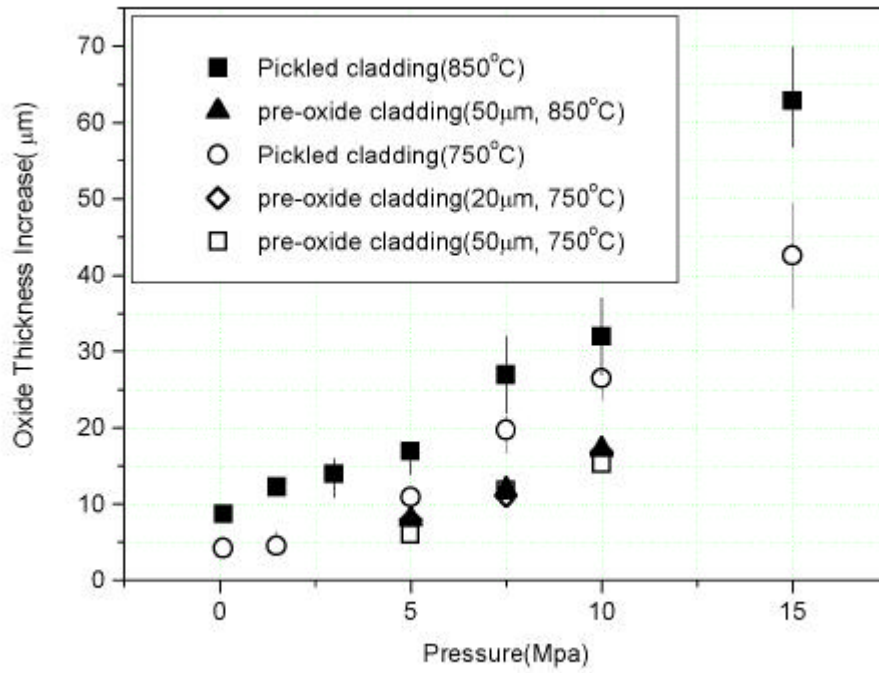


3 a) 750 °C, 75bar, 1500 h
 b) 750 °C, 5bar +70bar, 1500 h c) 700 °C, 1500 h



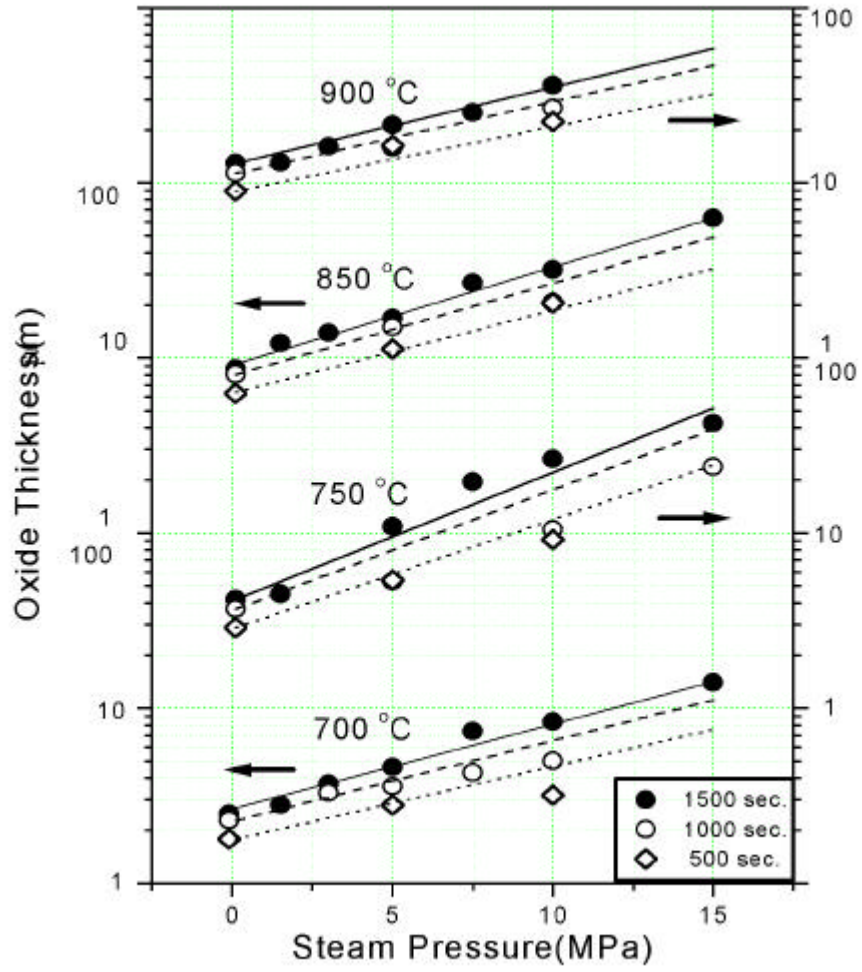


5



6

(1500)



7

1

가

()	n (bar ⁻¹)
700	1.53 x 10 ⁻³
750	2.31 x 10 ⁻³
800	2.27 x 10 ⁻³
850	1.76 x 10 ⁻³
900	1.38 x 10 ⁻³

