A study on the Improved Mechanical Properties of Nuclear Carbon Steel Piping through the Intercritical Heat Treatment



Abstract

In this study the intercritical heat treatment was applied to the SA106 Gr.C main steam line piping steel to investigate the improvement of toughness and the DSA sensitivity resulted from the heat treatment. Static strain aging (SSA) and J-R test were conducted to measure the material properties. The intercritical heat treatment gave a lower aging index than the as received condition at the lower aging temperature and activation energy determined by SSA experiment was 113.9 and 122.8 kJ/mol before and after heat treatment, respectively. Fracture tests show that the minimum value of J-R, Ji and dJ/da were observed at 289 when the load-load line displacement rate was 0.4mm/min, while there were minimum at 350 and 4.0mm/min. Compared to the as-received condition, temperature for the minimum fracture properties was shifted to higher temperature with the heat treatment.

2000

2

1.

가 가 (strain aging) [1]. 가 (USNRC) IPRIG (International Piping Integrity (dynamic strain aging; DSA) Research Group) 300 [2]. DSA , , . . DSA load drop, crack jumping 가 가 [1]. SA516 Gr.70 가 [2]. 288 가 , weld CLI , SA106 Gr.C 40% [3, 4]. 가 DSA 가 가 가 , (LBB) 2. 2.1 3, 4 SA106 Gr.C 1 • 950 1 • (furnace cooling) 2 760 40 (L-(annealing)), (L - C 가) .

	(decaburizing)		(СТ
가	. (ag	ging index)		가
フト 20mm	4mm	, J-R	ASTM E-1820	1T CT

2.1

	10	(AG-10TA,	Shimadzu Co.)			
-	(у	ield drop) J-R				
	- chart-r	ecorder		3	}	
	,					
			± 1.5	30		
	. J - R		COD gage		,	
		DCPD (Direct Cu	rrent Potential Drop)			,
50A	HP-DC power supp	ly				

2.2

			as-received	$2 \times 10^{-3} s^{-1}$	7%
		± 1	fan	dry ove	en
132, 150, 170	289				
			,	4mm	
•					

2.3 J-R

	AST	M E1820-96		(In stro	n 8501)	a/W=0.55
0.58						
10%	45 °	side- groo	ve			
200	, 가	289 ,	350	0.4, 4.0 mm/min		
		(J_i)			J	,
		(dJ/da) J-R		0.5 2	.5mm	1

3.

2				-	,	가	,
	,						가
	as-rec	eived					가
	[5].	2			가		'retained'
(phase)			,				

J - R .

1

3.1 가 as-received 1) , 가 , 3) , 2) Luder's strain 가 1). 4) (가 150 132 289 가 as-received , . 가 .

$$\frac{\Delta \sigma}{\overline{\sigma}} \equiv \Delta \sigma = (\sigma_y + \sigma_f), \quad \overline{\sigma} = (\sigma_y + \sigma_f)/2$$

$$\sigma_y = , \quad \sigma_f =$$
(1)

(aging index) 2 as-received 가 가 가 , 289 , 가 가 3 가 (over aging) . 가

as-received , as-received 가 3 .







Snoek () 1966 Hartley 7 7 7 Cottrell-Bilby [5], $\frac{\Delta\sigma}{\sigma} = K_1 + K_2 \left(\frac{Dt}{T_a}\right)^{2/3} ; (aging index) (2)$, as-received 7 113.9 122.8 [kJ/mol] (5). (Mn) [4].

3.2 가 가 가 J - R -6 7 0.4 mm/min7 350 J-R 가 가 J - R 289 가가 J - R . $4.0m \, m/min$ 가 가 289 350 J - R 가 . DSA가 가

 Ji as-received
 [10]
 8
 . 0.4mm/min
 ,
 Ji

 as-received
 Ji
 200
 289
 , 4.0

 mm/min
 Ji
 71 250
 350
 . Ji

[9]. 9 (dJ/da) 0.4 mm/mm가 가 dJ/da 가 289 가 . 가 가 dJ/da 4.0 mm/min350 Ji 289 350 . dJ/da .

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4.

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KSC SA516-Gr.70 / [2] (J - R), , 1998. 11. [3] 3/4 **RCS** Pipe (J - R), 1997. 9. [4] J.W. Kim, I.S. Kim, Nuclear Engineering and Design, Vol.172, 1997, p49. [5] J.S. Lee, et al., J of Korean Nuclear Society, Vol. 32, No.1, 2000, p77 87. [6] M.L. Weaver, et al., Materials Science and Engineering A, 192/193, 1995, p179 185. [7] H.E. Rosinger, Metal Science, Vol.9, 1975, p1 7. [8] S. Hartley, Acta Metallurgica, 1966, Vol. 14, p1237.

[9] P.S. Godavariti, Environmental Degradation of Materials in Nuclear Power Systems-Water Reactors, 1988, p105 109.

,

,

[10] LBB

1996. 8.

1 SA 106 Gr.C

С	Mn	Р	S	Si	Ni	Cr	Мо	V	Al	Cu	H_{ppm}
0.19	1.22	0.009	0.007	0.27	0.11	0.05	0.03	0.004	0.029	0.13	1.60



1 As received





(b)

, (a) as received, (b) heat treated







, (a) 132 , (b) 289

3





5 Hartley





(b)

_

, (a) $0.4m\,m/min$, (b) $4.0m\,m/min$





(b)

J-R , (a) 0.4mm/min, (b)

4.0 mm/min

7







