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1 : Top-down Approach TRAC-CSAU

Understanding of the Best-Estimate Methodology for LB-LOCA

Part-I: Top-down Approach and TRAC-CSAU

150

1998 [1] 가

가 [2] [3]

[4,5]

가

, 가

(LB-LOCA) 가 가¹

가 가

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()

(Separate Effect Test, SET)

(Integral Effect Test, IET)

TRAC-CSAU[5] [6]

¹ Bottom-up approach

²Top-down approach

Abstract

Since the law[1] for Large Break Loss of Coolant Accident(LB-LOCA) has been revised to allow the use of the Best-Estimate(BE) Methodology, The U.S. Nuclear Regulatory Commission(NRC) issued two independent positions for the old conservative evaluation model[2] and for the BE evaluation model[3] respectively. In this paper, by scrutinizing the U.S. regulatory position and the related studies[4,5], it is shown that a consistent regulatory principle is kept in both methodology. Following this principle, a methodology can be suggested to select a certain code model or correlation and to quantify their ranges for the concerned important uncertain phenomena or process. The key point of the BE-methodology is that the identified important phenomena or processes are to be evaluated to quantify the uncertainties in the LB-LOCA scenario, instead of evaluating the individual models and correlations. In doing so, however, a certain model or group of models should be selected and their uncertainties should be quantified. But, their role should be understood as the ‘representative’ parameters for the concerned phenomena or processes. Accordingly, the range of the selected uncertainty parameter should be confirmed based on the well-scaled experiments. With the above viewpoint, some solutions are suggested for the many questions[6] concerning the TRAC-CSAU[5] methodology.

1.

TRAC-CSAU 가 , ,
 . -1 , TRAC-CSAU
TPG group[6] . TRAC-CSAU
 , 7 . ,
“ 가?” .
TPG group ;

“It is not possible nor necessary to address each potential source of uncertainty in an equal manner. One of the strengths of the methodology is its ability, based on expert evaluation of experimental evidence, to prioritize the sources of uncertainty for further analysis and treat them accordingly”

TRAC-CSAU [5] NUREG-1230[4]
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 가?’ , ‘
 가 가?’ .
LB-LOCA

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2. Top-down Approach; LB-LOCA

Top-down approach LB-LOCA .

LB-LOCA LB-

LOCA 가 ,

LB-LOCA .

LB-LOCA .

top-down approach .

2.1 Appendix-K Top-down Approach;

Appendix-K top-down approach 가

NUREG-1230[4] ;

“Because extensive directly applicable experimental data did not yet exist in 1972 for use in computer code development or in assessing the predictive capabilities of these codes for key portions of the Light Water Reactor (LWR) response to LOCAs, large uncertainties existed in predictions of these transients. Accordingly, sufficiently conservative assumptions were used in developing 50.46 and Appendix-K in 1974 to provide assurance that Emergency Core Cooling System(ECCS) criteria would be satisfied even in the unlikely event that worst-case uncertainties prevailed.”

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2

. Appendix-K 가 required features -3

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Appendix-K 가 required features

가 required features /

LB-LOCA

Appendix-K required features 가 /

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2.2 가 Top-down approach
 LB-LOCA 가 가 [3]
 . -3 Reg Guide1.157 . , -1
 -2 . -3
 Appendix-K 가 , LOCA . -4
 가 . -3
 Appendix-K . -3
 -4 가 .
 -2 NUREG-1230[4] 가 .
 NUREG-1230 ;

“There can be two method to evaluate a best estimate code. In the bottom-up approach, each model and all the closure relations in a code are examined and assessed in a uniform function. Sensitivity studies are performed on every single model to assess its effects on calculated results. Although this approach is rigorous, it is definitely impractical in view of the number of calculations that would be required.

In the top-down approach, one identifies significant phenomena that have influence on the overall results for a scenario or for a distinct class of scenarios. The capability of the code to calculate these significant phenomena is assessed against test data. Finally, sensitivity studies are performed on parameters and/or models that affect the significant phenomena. It is evident that the top-down approach has several attractive features, for example, the reduced number of sensitivity calculations. However, it still has important shortcomings, that is, it does not offer a method to address the questions related to scaling and to compensating errors among others. CSAU method, which removes these and other shortcomings ...”

	bottom-up approach	top-down approach
(-4). CSAU	top-down approach	QA documents
SET IET 가		
가	TRAC-CSAU	[5].
TRAC-PF1[7]	4	LB-LOCA
.	/	-1 . ,
/	(-2),	Appendix-K
.	required features	uncertainty parameters

top-

down approach 가 . , / 가
Appendix-K /
/
가
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가?

3.

CSAU 가 .
TRAC-CSAU phenomena/process identification and ranking

-1 break discharge coefficient, fuel parameters, heat transfer coefficient, minimum boiling temperature, pump, steam binding, ECC bypass, dissolved nitrogen
heat transfer coefficient minimum boiling temperature , stored energy
and fuel response 가 가 clad surface heat transfer

가 가 [-1]. 가
heat transfer coefficient minimum boiling temperature 가
fuel response .

. , Appendix-K 가 fuel response no return
to nucleate, no return to transition boiling, hot channel to be less than one assembly size, FLECHT
correlation to be used for 1 in/sec, steam cooling to be used for 1 in / sec required
features (-2).

가 . , Reg. Guide Position 4.2 (-4) SET IET 가 ;

“Code uncertainty should be evaluated through direct data comparison with relevant integral systems and separate effects experiments at different scales. In this manner, an estimate of the uncertainty attributable to the combined effect of the models and correlations within the code can be obtained for all scales and for different phenomena. Comparisons to a sufficient number of integral systems experiments from different facilities and different scales, should be made to ensure that a reasonable estimate of code uncertainty and bias has been obtained....”

가 . SET
IET 가 가 code uncertainty bias IET
Appendix-K required features

가 /
 . TRAC-CSAU heat transfer coefficient
 minimum boiling temperature 가 .

3.1. heat transfer coefficient

fuel response , flow regime map,
 , TRAC-CSAU

Appendix-K required feature
 . TRA-CSAU 가
 single heated tube [7] , 가 .

$$h(t) = h(W(t), \mathbf{a}(t), K, c_p, \dots, t)$$

h 가 t
 가 .

$$h(t) = h_i(W(t), \mathbf{a}(t), K, c_p, \dots) \quad t_{i-1} < t < t_i$$

i (heat transfer regime) .

1. $h_i(t)$ single tube test[8]

가 가 가

2. 가
 W \mathbf{a}

가 .
 3. 가

가 , droplet entrainment, droplet evaporation
가
4.
grid ,

5. 가

가 ,
fuel response ‘ ’ , , INEL Post-CHF
SET IET

가 가 code uncertainty bias IET

TRAC-CSAU Reg. Guide

Reg. Guide

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가 ,

가

가

가

power-to-volume

가

THTF[9]

. THTF ,

, film boiling

transition boiling

grid

가 , distortion

FLECHT-SEASET[10]

가

가

IET 가

Reg. Guide

3.2 TRAC-CSAU heat transfer coefficient

가 가 1 , PIRT process

가 SET IET

가 , fuel

response , THTF 가 가

가 fuel response . THTF

가 .

TRAC-CSAU , break discharge coefficient , break flow

, Mraviken[11]

가 . Mraviken LB-LOCA

critical flow scale .

/ 가 . fuel response

TRAC-CSAU ,

Appendix-K decay heat no return to nucleate boiling

fuel response 가 가

THTF

SET 가 single tube test 가 .

TRAC-CSAU SET IET .

4.

TRAC-CSAU

fuel response

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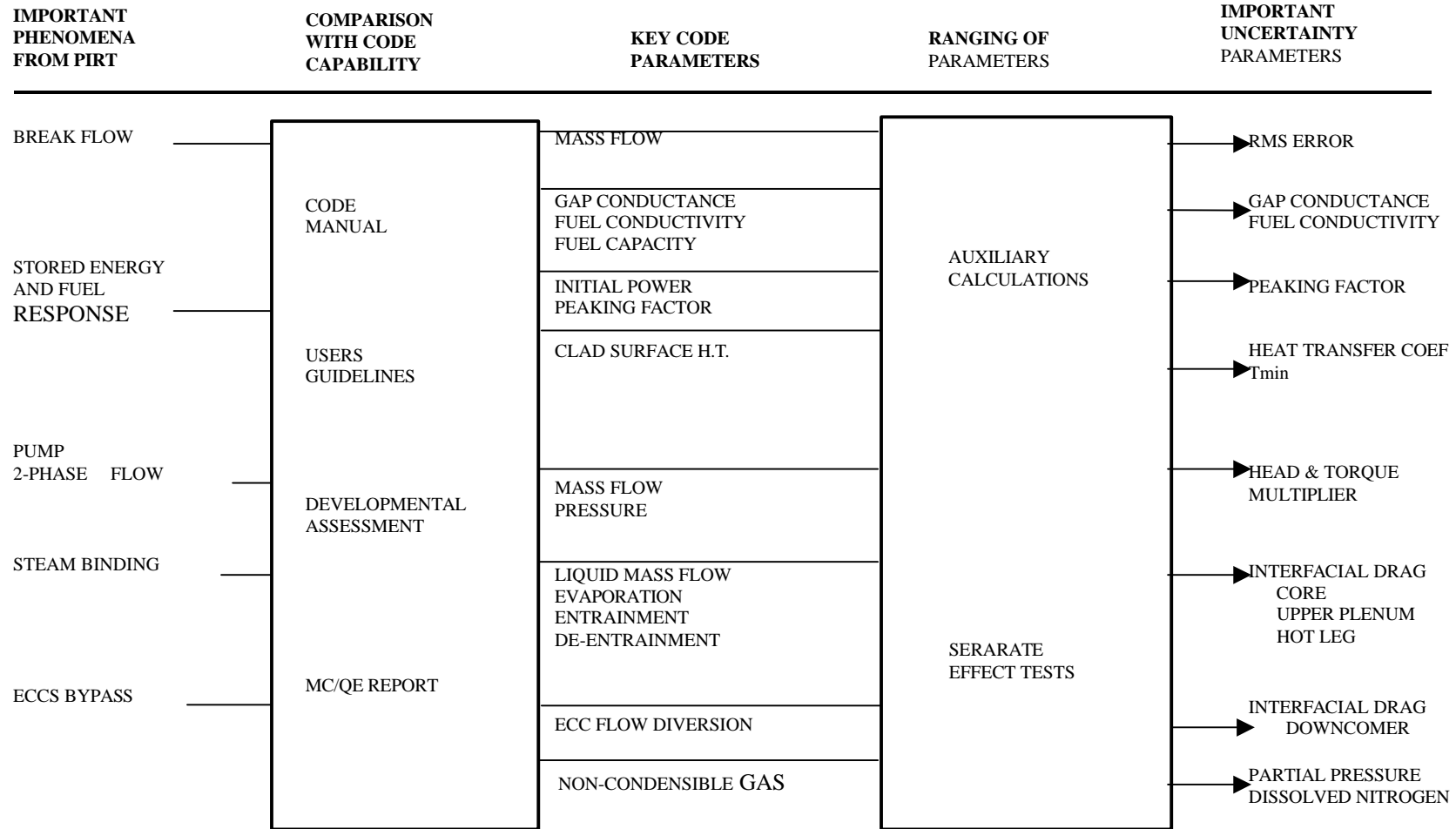
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SET IET

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1. United States Code of Federal Regulations, Title 10, Section 50.46, "Acceptance Criteria for Emergency Core Cooling Systems for Light Water Reactors", 1988.
2. APPENDIX-K to Part 50—ECCS Evaluation Models, I. Required and Acceptable Features of Evaluation Models; Regulations—U.S. Nuclear Regulatory Commission
3. Regulatory Guide 1.157, "Best-Estimate Calculations of ECCS Performance", U.S. Regulatory Commission, May 1989.
4. Division of Reactor and Plant Systems, "Compendium of ECCS Research for Realistic LOCA Analysis". NUREG-1230, 1987.
5. B. Boyack, et. al., "Quantifying Reactor Safety margins". NUREG/CR-5249, 1989.
6. G. E. Wilson, et. al., "TPG response to the foregoing letters-to-editor". Nucl. Eng. and Des. V132, pp431, 1992.
7. D. R. Liles et. al., "TRAC-PF1/MOD1, An advanced Best-Estimate Computer Program for Pressurized Water Reactor Analysis". NUREG/CR-3858, July 1986.
8. R. C. Gottula et. al., "Forced convective, Non-equilibrium, Post-CHF heat transfer experiment data and correlation comparison report". NUREG/CR-3193, March 1985.
9. V. D. Clemens et. al., PWR Blowdown Heat Transfer Separate-Effects Program --- Thermal-Hydraulic Test Facility Experimental Data Report for Test 160, NUREG/CR-0730, June 1980.
10. M. J. Loftus, et. al., PWR FLECHT-SEASET Unblocked Bundle, Forced and Gravity Reflood Task Data Report, NUREG/CR-1532, September 1981.
11. Studsvik Energi teknik AB, "The Marviken FullScale Critical-Flow Tests", Final report Vol 1-35, EPRI/NP-2370, December 1982.

-1 TRAC-CSAU



-1. TRAC-CSAU

No.	Questions on
1	Generality of CSAU
2.	Practicality of CSAU
3	Use of Engineering Judgement in application of CSAU
4	Use of Biases in application of CSAU
5	Use of frozen Code Version having Model Deficiency
6	Quantification of uncertainty introduced by user controlled Variations, particularly Nodalization
7	Elimination of many thermal-hydraulic phenomena as sources of Uncertainty
8	Validity of comparing the CSAU results to the world' s PCT data bases
9	Validity of the Use of Supplemental Fuel Rods
10	Scalability of Code
11	Statistics

-2. Appendix-K Required Features and TRAC-CSAU

Phenomena/Processes	Required features	TRAC-CSAU Uncertainty Parameters
A. Sources of Heat B. Fuel Rod Model C. Blowdown Phenomena 1. Break Flow a. spectrum b. discharge model c. EOB d. Noding near break 2. Frictional Pressure Drop 3. Momentum Equation 4. Critical Heat Flux 5. Post-CHF HTCorrelation 6. Pump Modeling 7. Core Flow Distribution D. Post-Blowdown Phenomena 1. Single Failure Criterion 2. Containment Pressure 3. Reflood rate calculation 4. Steam condensation 5. RF H.T and steam cooling	A. Decay Heat; ANS-71 plus 20% B. Swelling and rupture Gap conductance C. Blowdown Phenomena 1. Break Flow a. spectrum b. Moody Model c. EOB d. Split break 2. Realistic Frictional Pressure Drop 3. Momentum Equation 4. No return to Nucleate 5. No return to Transition Boiling 6. Two-phase Pump Data 7. Hot Channel<One Assembly Size D. Post-Blowdown Phenomena 1. Single Failure Criterion 2. Minimum Back Pressure 3. High entrainment rate 4. No condensation at pipes 5. FLECHT correlation > 1 in/sec Steam cooling < 1 in/sec	Peaking factor Gap conductance Discharge coeff. ECC bypass Tmin Heat Transfer Coeff Pump Degradation Steam binding Tmin Heat Transfer Coeff

-3. Summary of Regulatory Guide 1.157

Regulatory Position 1; General Attributes in Best-Estimate Calculations

Best-Estimate Model; realistic calculation of experiment predict mean value of experiments
 Unaccounted-for Model; treated as a bias in overall uncertainty do not include the bias in the analysis
 Range of Models; use within the applicable range if extrapolated, uncertainty evaluation
 Best-Estimate Code; predict the important phenomena
 SET and IET; determine overall uncertainty and bias / IET; confirmation of Best-Estimate Code
 Conservatism; simplification leads little or no effect; only upper bound of model is known
 Bias is acceptable

Regulatory Position 2; Special Considerations for Best-Estimate Calculations

NUREG-1230 Compendium guides the Best Estimate Methodology

Uncertainties of features; included in overall uncertainty calculation

2.1 Basic Structure of Codes

- 2.1.1 Numerical Methods; Noding Sensitivity to be done
- 2.1.2 Correlational Models

Regulatory Position 3; Best-Estimate Code Features

- 3.1 Initial and Boundary Condition and Equipment Availability
- 3.2 Sources of Heat During a LOCA
- 3.3 Fuel Rod Parameters
- 3.4 Blowdown Phenomena
 - 3.4.1 Break Characteristics and Flow /3.4.2 ECC Bypass
- 3.5 Noding near the Break and ECCS injection Point
- 3.6 Frictional Pressure Drop
- 3.7 Momentum Equation
- 3.8 Critical Heat Flux
- 3.9 Post-CHF blowdown Heat Transfer
- 3.10 Pump Modeling
- 3.11 Core Flow Distribution During Blowdown
- 3.12 Post-Blowdown Phenomena
 - 3.12.1 Containment Pressure
 - 3.12.2 Calculation of PB Th for PWR
 - 3.12.3 Steam Interaction with ECC in PWR
 - 3.12.4 Post Blowdown Heat Transfer for PWR
- 3.16 Other Features
 - 3.16.1 Completeness
 - BE code should be complete
 - 3.16.2 Data Comparisons
 - Individual Models should be compared with Experiments

Regulatory Position 4; Estimation of Overall Calculation Uncertainty

- 4.1 General;
 - Definition of Uncertainty;
 - Code Uncertainty; Combined uncertainty accounting the individual models and correlations
 - Overall Calculation Uncertainty;
 - Code Uncertainty Plus Uncertainties from the Various Sources
 - A completely rigorous mathematical treatment is neither practical nor required.
 - Approximations and assumptions
- 4.2 Code Uncertainty
 - Evaluated by direct comparison with relevant IET' s and SET' s
 - Separate Uncertainties for Blowdown and Reflood
 - Justification of separate uncertainty treatment
- 4.3 Other source of Uncertainty
 - 4.3.1 Initial and Boundary Conditions and Equipment availability
 - 4.3.2 Fuel Behavior
 - 4.3.3 Other Variables
- 4.4 Statistical Treatment of Overall Calculation Uncertainty

-4. Bottom-up and top-down approach for code evaluation

Approach	Range of assess	Sensitivity range	Merits	Demerits
Bottom-up	Single model	Every single model	rigorous	impractical
Top-down	Phenomena	Parameter affecting phenomena	economic	Scaling error Compensating error
