

Optimal Design of Hybrid Vane in a Nuclear Fuel Bundle by the Flow Analysis

150

/ 30° - 40° 0° - 45° 가
 40° 35° 가 가
 가

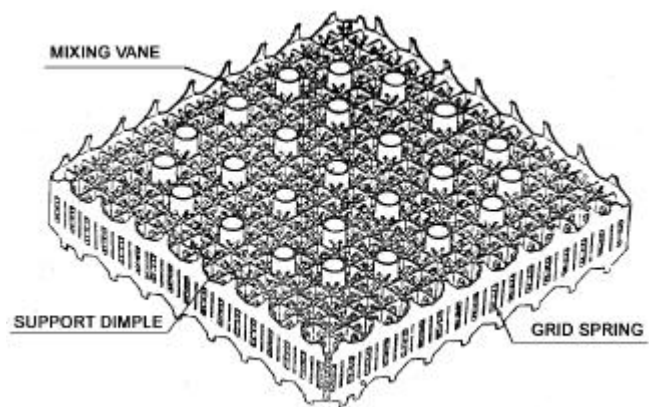
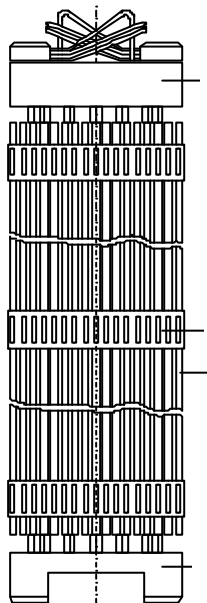
Abstract

A computational fluid dynamics (CFD) analysis was performed to propose an optimal design of hybrid vane in a nuclear fuel bundle. The hybrid vane is a new coolant-mixing device under development by Korea Atomic Energy Research Institute, which consists of two sets of primary and secondary vanes. To maximize the coolant mixing by the hybrid vane, its size and vane angle must be optimized. The vane angle, defined as the angle bent from the axial flow direction, changed from 30° to 40° and from 0° to 45° for the primary and secondary vanes, respectively. Effect of the secondary vane width was examined for three sizes. The swirl and crossflow mixing factors were estimated from the predicted velocity distributions in the fuel bundle. Pressure drop and turbulence increase due to the hybrid vane was also evaluated. The optimal vane angles are judged to be 40° and 35° for the primary and secondary hybrid vanes, respectively. The secondary vane width should be rather small to increase the crossflow mixing but not to significantly reduce the swirl mixing.

1.

가 (subchannel) (grid spacer)
 (1).
 (critical heat flux)
 가 가
 (swirl) (crossflow)
 (computational fluid dynamics)

Karoutas ⁽¹⁾ 가 3
 CFD
 CFDS - FLOW3D
 CFD
 Imaizumi ⁽²⁾ 가
 In ^(3,4)
 CFD
 Ikeda Hoshi ⁽⁵⁾
 (hybrid vane)가 ⁽⁶⁾
 가 CFD



1. 가

2.

2.1

2

4

2

(primary vane)

(secondary vane)

가

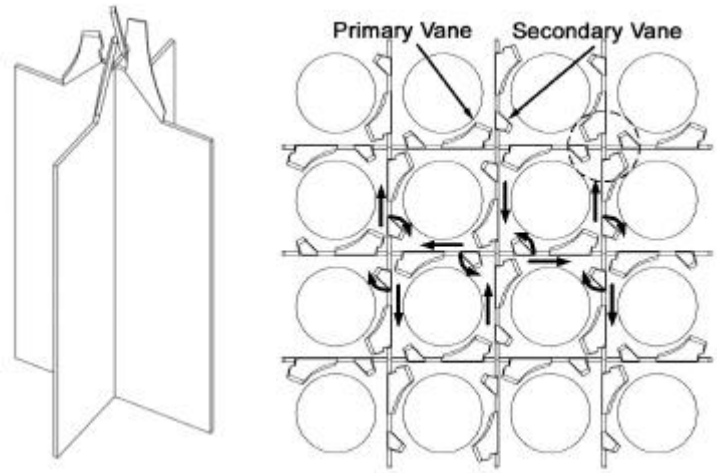
가

가

가

가

40°



2.

2.2

30 mm

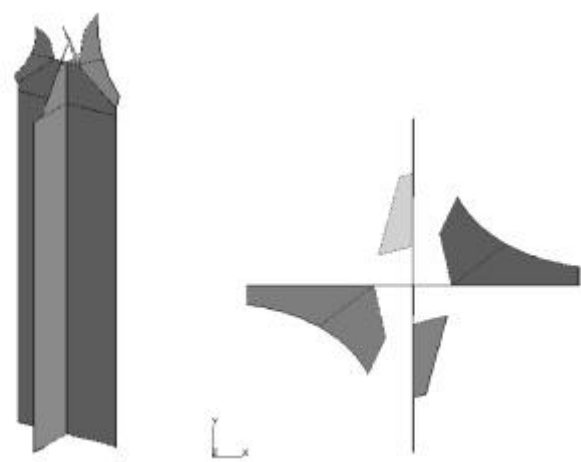
(,)

(D) 9.5 mm (P) 12.8 mm 3 가

120 3 (structured grid)

32 , (gap) 20 200 3

206000 4



3.

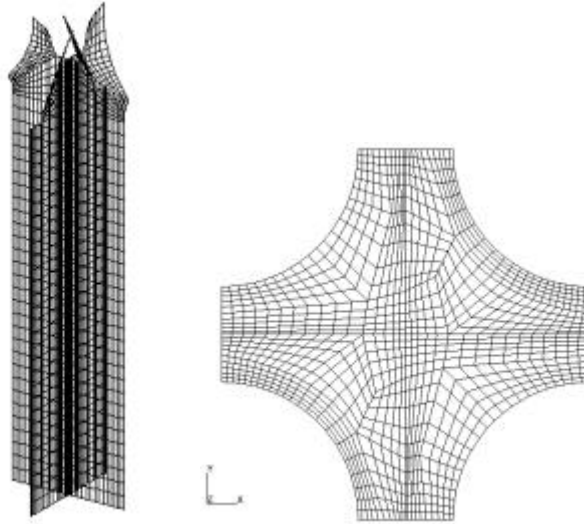
가

가

가

가

(no - slip)



4.

2.3

가

CFD

CFX - 4⁽⁷⁾

3

Launder Spalding

$k - e$

(8)

$k - e$

(algebraic multi - grid)

(under - relaxation)

(residual)

0.05%

가

7000

HP9000 C200 C180(PA8000 CPU, 1.0GB RAM)

(V_{bulk})

7.0

m/s (Re=87000)

(D_h) 12.5 mm

(q_1)

가 40°

$30^\circ, 35^\circ, 40^\circ$

(q_2) 0° (

가

), $20^\circ - 45^\circ$

(w)

(

x 2)

($w = w_0$)

($w = 1.2w_0$)

($w = 0.8w_0$)

가

3.

3.1

가

5

가 (z=2D_h)

z=5D_h

z=10D_h

z=20D_h

z=5D_h

가

가

가

가

가 (V_{axial})

(V_{lateral})

(V_{cross})

가

가

(x=0)

가

가

z=30D_h

가

가 (z

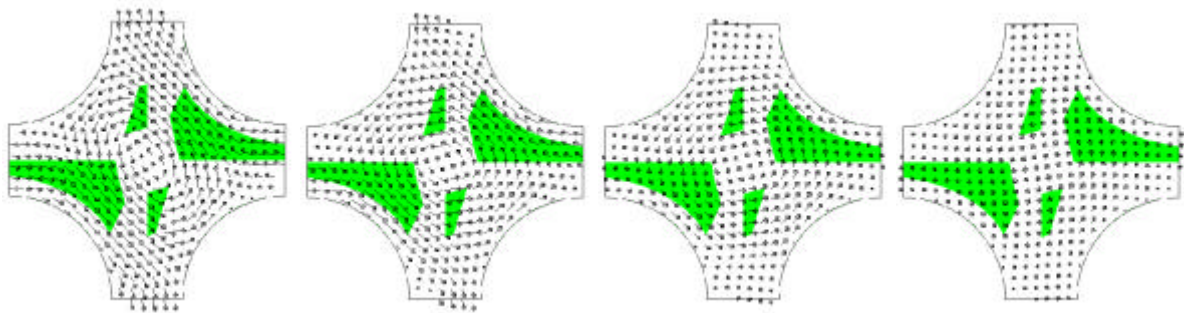
< 5D_h)

가

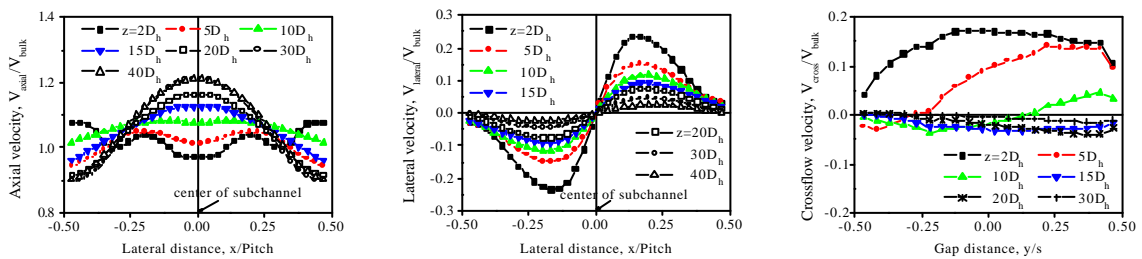
(+)

z=5D_h

z=15D_h



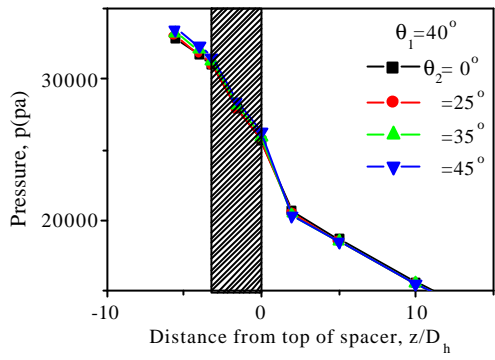
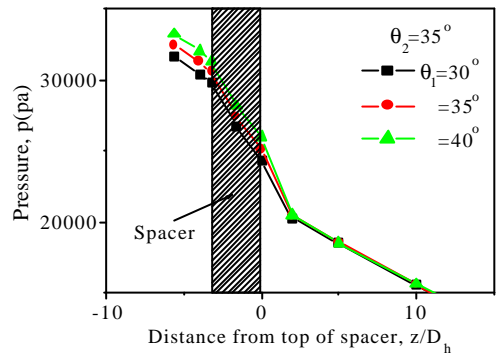
5. ($\theta_1 = 40^\circ, \theta_2 = 35^\circ, w = 0.8w_0$); () z=2D_h, 5D_h, 10D_h, 20D_h



6. ($\theta_1 = 40^\circ, \theta_2 = 35^\circ, w = 0.8w_0$)

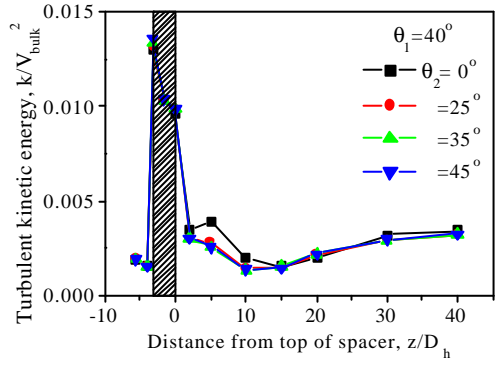
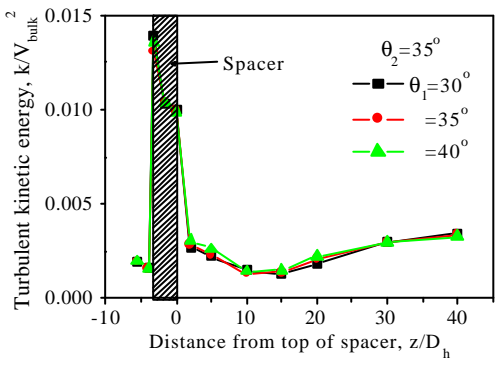
3.2

가
가
가
가
가
가
가
가



7.

; () , ()



8.

, ()

; ()

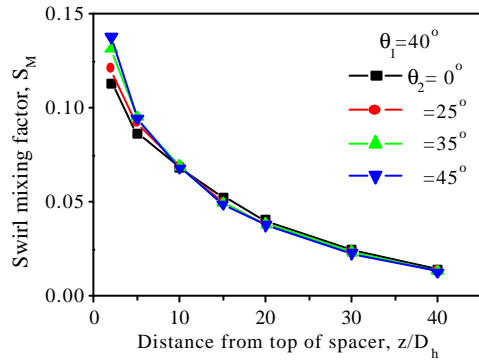
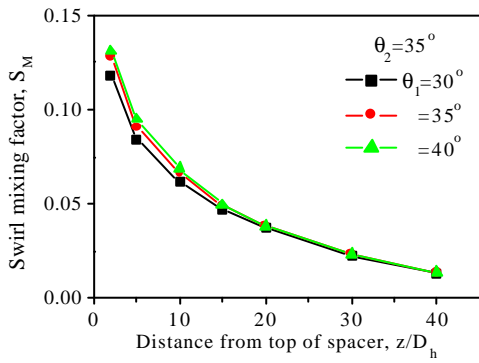
3.3

가
(F_{CM})
가
(S_M)

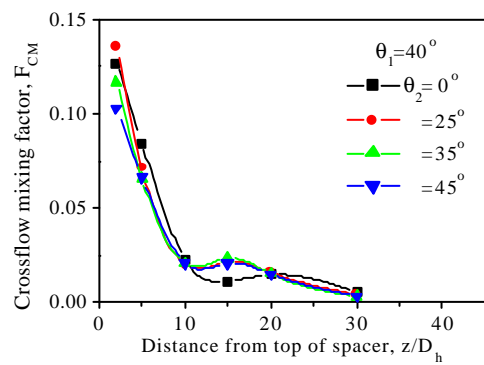
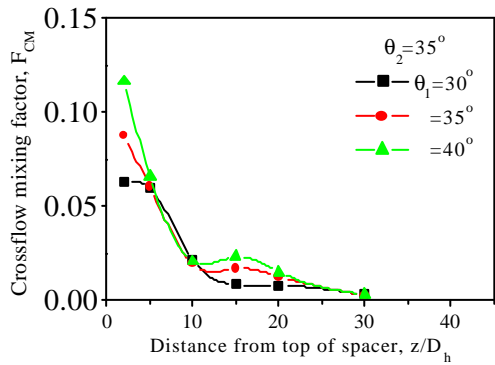
$$S_M = \frac{\int r^2 V_{lateral} V_{axial} dr}{R_S \int r V_{axial}^2 dr}, \tag{1}$$

$$F_{CM} = \frac{1}{s} \int \frac{|V_{cross}|}{V_{bulk}} dy. \quad (2)$$

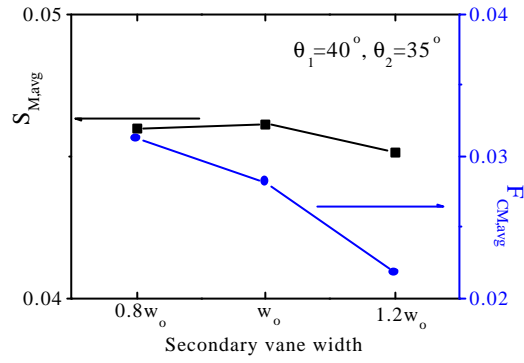
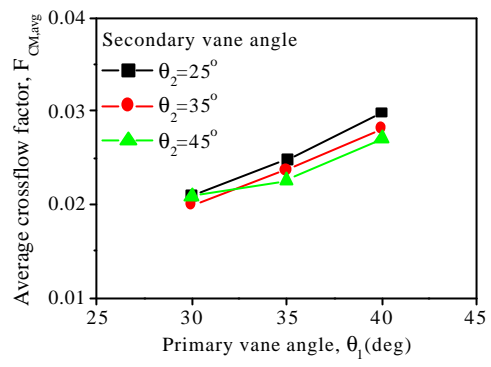
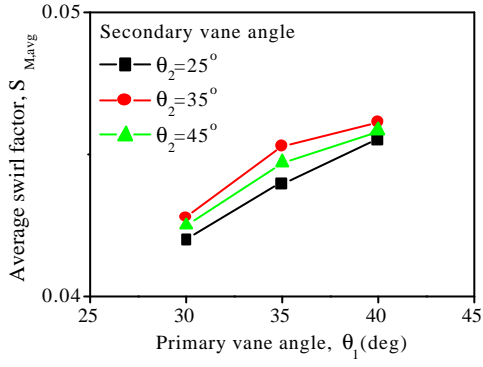
r , R_s
 . (2) s
 9
 가 가
 . (z>30D_h) 가 가
 가 가 가 35° 가
 , 10 가 가
 . z=15D_h 가 가
 z=5D_h - 10D_h 가
 z=15D_h 가 가
 가 가 가 (z<5D_h)



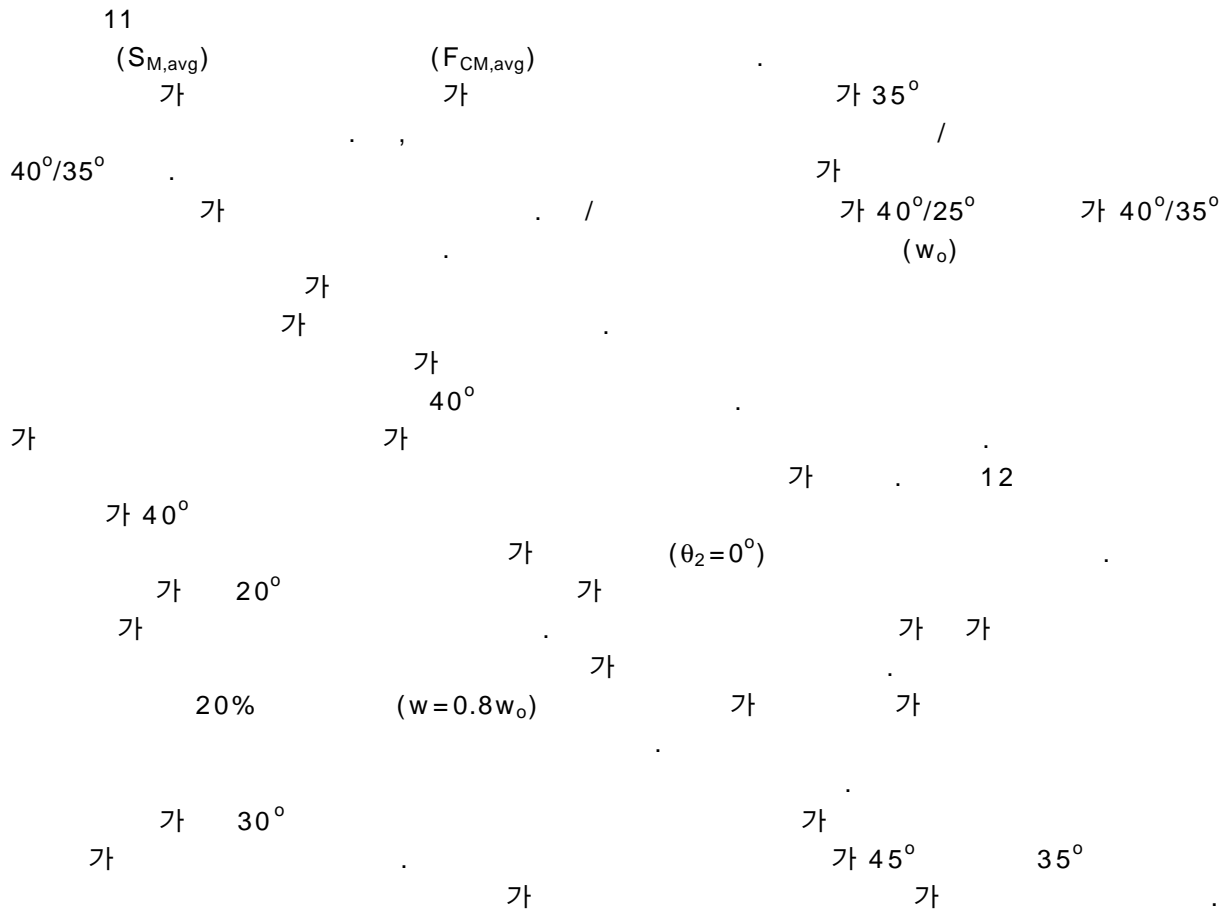
9. ; ()
 , ()



10. ; ()
 , ()



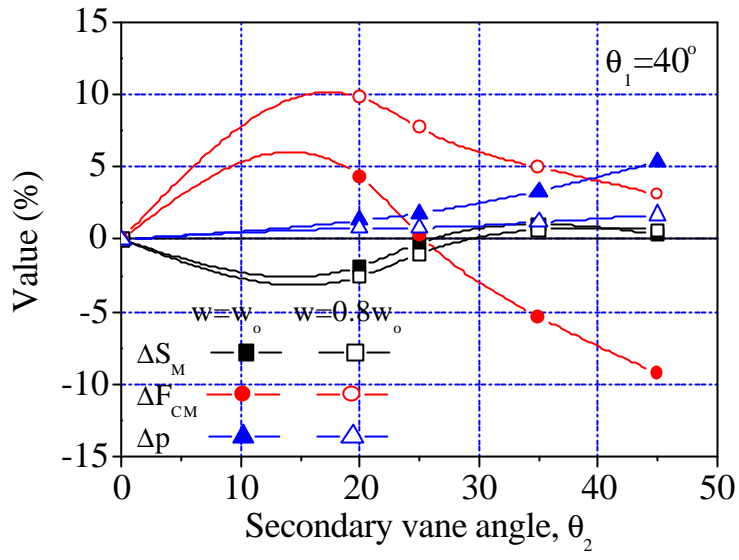
11. $(S_{M,avg})$ $(F_{CM,avg})$



12

35°

20%



12.

4.

30° - 40° 0° - 45°

가 가 가

가 가 가 가 가 가

가 가 가 가 가 가 가 가

(w₀) 35° 20%

- D
- D_h
- F_{CM}
- S_M
- P
- w
- w₀
- θ₁
- θ₂

$$(\sqrt{2}P - D)$$

- (1) Karoutas Z., Gu C.Y. and Scholin B., 1995, "3-D flow analyses for design of nuclear fuel spacer," *Proc. of the 7th Int. Meeting on Nuclear Reactor Thermal-Hydraulics*, New York, United States, September 10-15.
- (2) Imaizumi M., Ichioka T., Hoshi M., Teshima H., Kobayashi H., and Yokoyama T., 1995, "Development of CFD method to evaluate 3-D flow characteristics for PWR fuel assembly," *Trans. of the 13th International Conference on SMiRT*, Porto Alegre, Brazil, August 13-18.
- (3) In W. K., 2001, "Numerical study of coolant mixing caused by the flow deflectors in a nuclear fuel bundle," *Nuclear Technology*, **134**, pp. 187-195.
- (4) In W. K., Oh D. S. and Chun T. H., 2001, "Flow analysis for optimal design of mixing vane in a PWR fuel assembly", *J. of KNS*, **33**(3), pp. 327-338.
- (5) Ikeda K. and Hoshi M., 2001, "Development of Mitsubishi high thermal performance grid 1 – CFD applicability for thermal hydraulic design," *9th Int. Conference on Nuclear Engineering*, Nice, France, April 8-12.
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- (8) Launder B. E. and Spalding D. B., 1974, "The numerical computation of turbulent flows," *Computational Methods in Applied Mechanics and Engineering*, **3**, 269.