

PLUS7 가

The Thermal Performance Evaluation for PLUS7 Fuel

493

PLUS7 가 .
 PLUS7 KCE-1 . PLUS7
 , PLUS7 가 12.8%
 , 5&6 PLUS7
 AOPM(Available Over Power Margin) , PLUS7
 14.8% 가 가 .

Abstract

The PLUS7 Fuel was developed with advanced mixing vane grids for Korean Standard Nuclear Plants(KSNP). A series of CHF tests were performed for the PLUS7 fuel and the KCE-1 CHF correlation was developed, based on the CHF data. It was confirmed that the thermal performance of the PLUS7 fuel is increased by 12.8% in average compared to that of KSNP standard(STD) fuel, through the direct comparison of CHF data for the PLUS7 and the STD fuels. Also, it was evaluated that the thermal performance of the PLUS7 is 14.8 % higher than that of the STD fuel in the standpoint view of the available over power margins for the UCN 5&6 core.

1.

PLUS7 . PLUS7 가 . PLUS7
 KCE-1
 (Critical Heat Flux Correlation) .
 PLUS7
 KCE-1 . ,
 PLUS7
 5&6 PLUS7 AOPM(Available
 Over Power Margin) PLUS7 가 .

2. PLUS7

PLUS7 / , , , ,
 , 4 1 ZIRLO , ,
 16x16
 .(1, 2) 55,000MWD/MTU
 Zry-4 ZIRLO .
 0.374
 (0.382)
 Contour / . 가
 / -
 718 가 .
 1/2 가 , 가
 가 ,
 가 .

3. KCE-1

3.1.

PLUS7 ,
 (HTRF,
 Heat Transfer Research Facility) [1, 2]. (test section)
 6x6 , 3 가 9
 가 가 (typical
 channel) (thimble channel)
 가 (4, 5).
 (cold wall effect) 가 가
 6x6 .
 가 가 가
 가 가 가
 가 가 가
 , 가 가
 , 6x6
 6x6 PLUS7 5x5 가 .
 1 ,
 2 .

3.2.

PLUS7
CE-1 [3]

KCE-1

$$q_{CHF,U} = \frac{B_1(d/d_m)^{B_2} \left[(B_3 + B_4 P)(G/10^6)^{(B_5+B_6)} - (G/10^6) c h_{fg} \right]}{(G/10^6)^{(B_7 P + B_8 (G/10^6))}} \quad (1)$$

$q_{CHF,U}$ critical heat flux for uniform axial power, *MBtu/hr-ft²*
P pressuer, *psia*
d heated diameter of subchannel, *inches*
d_m heated diameter of matrix subchannel, *inches*
G local mass velocity at CHF location, *lbm / hr-ft²*
c local coolant quality at CHF location, decimal fraction
h_{fg} latent heat of vaporization, *Btu/lbm.*

TORC [4] PLUS7
가

가

가

가

Tong F [5]

KCE-1

CE-1

B₂ CE-1

1/5

가

KCE-1

6 9

6 KCE-1

가

가 1

7, 8, 9

(local quality)

(M/P)

7 1400 psia

M/P

1

KCE-1

8 9

M/P

1

가

PLUS7

KCE-1

System Pressure	[<i>psia</i>]	1395	to	2415
Local Quality	[-]	-0.150	to	0.275
Local Mass Velocity	[<i>Mlbm / hr-ft²</i>]	0.85	to	3.15
Heated equivalent diameter,	[<i>in</i>]	0.4976	to	0.7152

4.

가

4.1.

PLUS7

가

3.1

PLUS7

$$Increase\ Power = \frac{P_{KCE1} - P_{CE1}}{P_{CE1}} \times 100 \quad (2)$$

, P : CHF rod average heat flux

3.8%, 24.0%, 12.8% 10 PLUS7 가

가

4.2.

5&6

5&6

PLUS7

가

KCE-1 CE-1

DNBR

AOPM

Operating Condition

Pressure,	psia	2250
Inlet Temperature,	°F	564.5
Flow Rate	%	100.0 of system flow rate
DNBR	-	1.30

Available Over Power Margin

STD Fuel	%	20.48
PLUS7 Fuel	%	35.25

, PLUS7 14.8% 가

5.

PLUS7

가

, PLUS7

가

PLUS7

KCE-1

가

. PLUS7

가

, PLUS7
 5&6 PLUS7
 AOPM PLUS7 가
 3.8%, 24.0%, 12.8%
 AOPM PLUS7
 14.8% 가 가 .

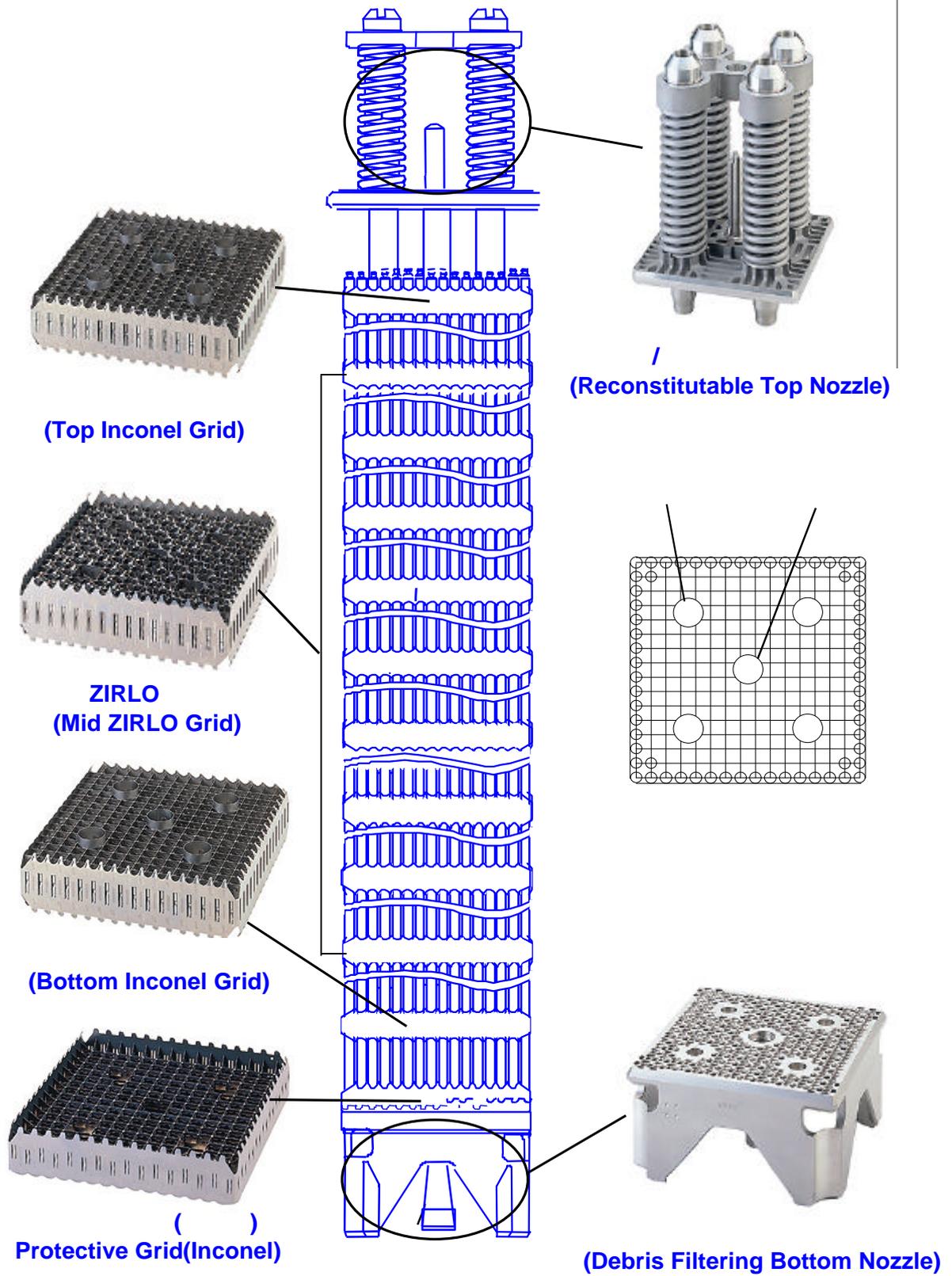
1. CU-HTRF-2001-W1010, "Critical Heat Flux Tests on PWR Fuel Assemblies for Westinghouse Electric Company Test No. 101.0", August 2001.
2. CU-HTRF-2001-W1020, "Critical Heat Flux Tests on PWR Fuel Assemblies for Westinghouse Electric Company Test No. 102.0", August 2001.
3. CENPD-162-P-A, "C-E Critical Heat Flux, Critical Heat Flux Correlation for C-E Fuel Assemblies with Standard Spacer Grids, Part 1 Uniform Axial Power Distribution", September 1976.
4. CENPD-161-P-A, "TORC Code, A Computer Code for Determining the Thermal Margin of Reactor Core", April 1986.
5. Tong, L. S., "Boiling Crisis and Critical Heat Flux", U. S. Atomic Energy Commission, 1972.

1. PLUS7

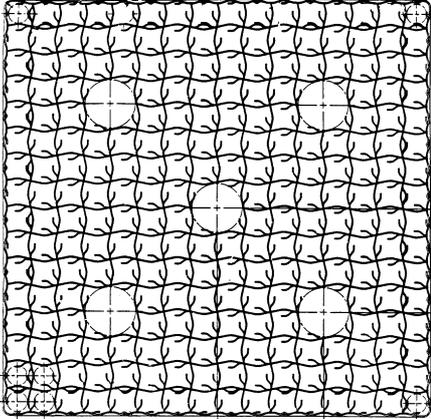
Test Class	Fuel Type	Rod Diam.[in]	Rod Pitch [in]	Heated Length[in]	Grid Spring[in]	Guide Tube	GT Diam.[in]	Axial Shape	Grid Material
Typical	16 x 16	0.374	0.506	150.0	15.7	No	N/A	1.475 cos	ZIRLO™
Thimble	16 x 16	0.374	0.506	150.0	15.7	Yes	0.980	1.475 cos	ZIRLO™

2.

, <i>psia</i>	1400 ~ 2490
, <i>Mlbm/hr-ft²</i>	0.90 ~ 3.70
, <i>°F</i>	250 ~ 637

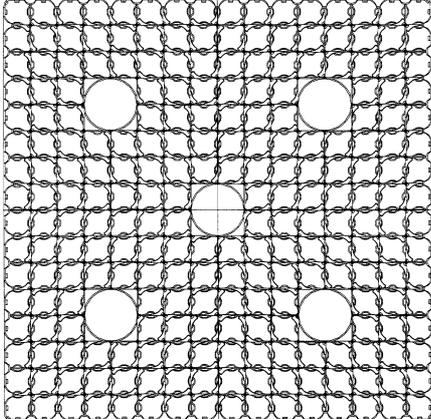


KSNP

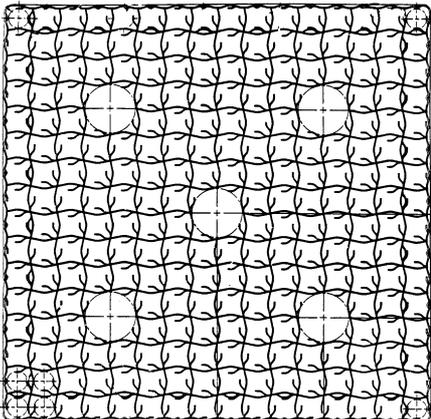


<

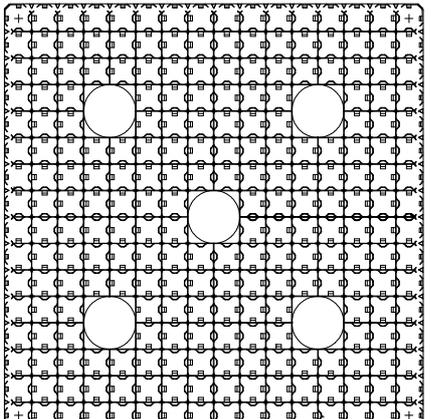
PLUS 7



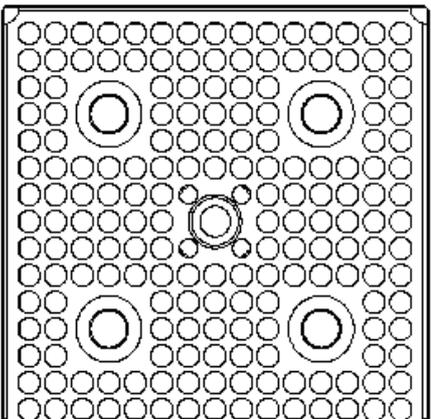
>



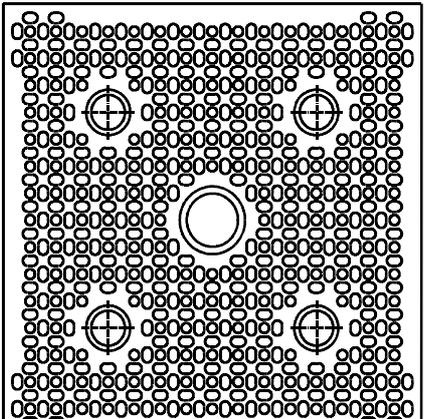
<



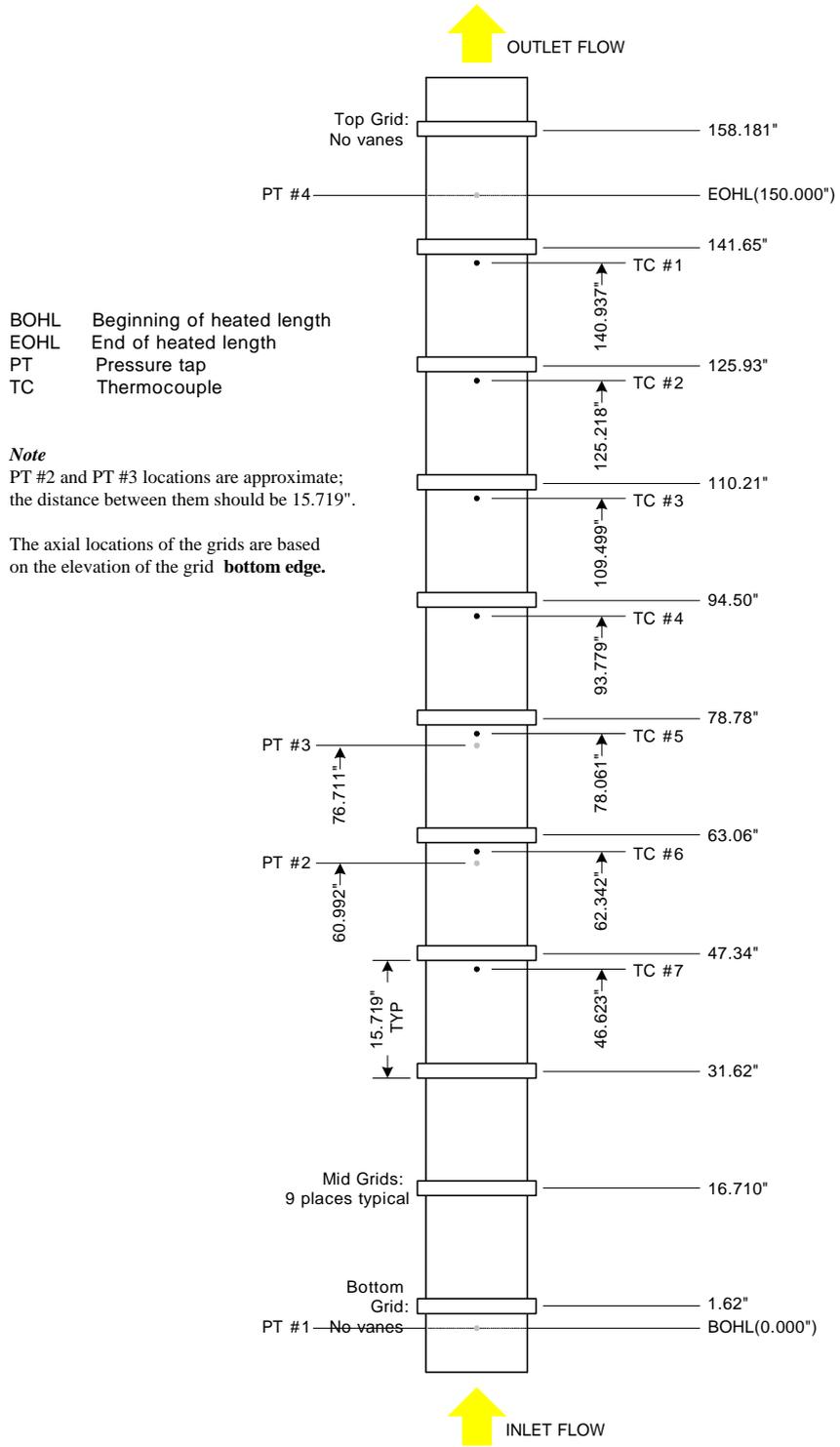
>



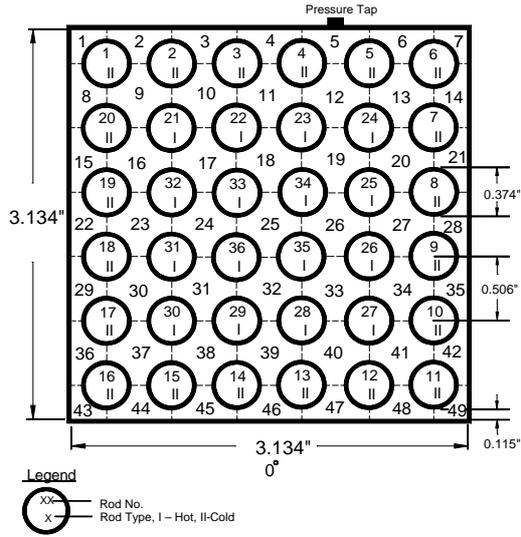
<



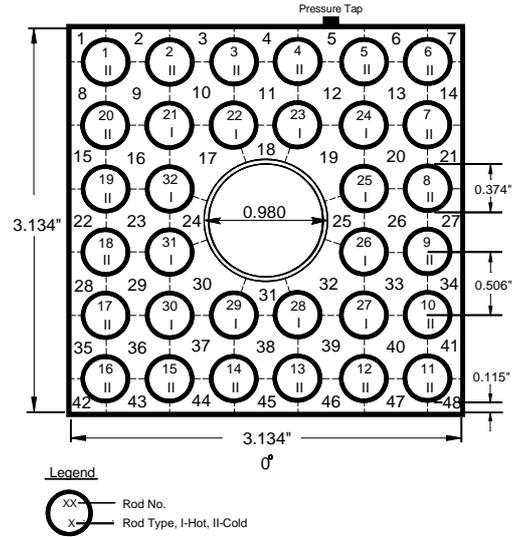
>



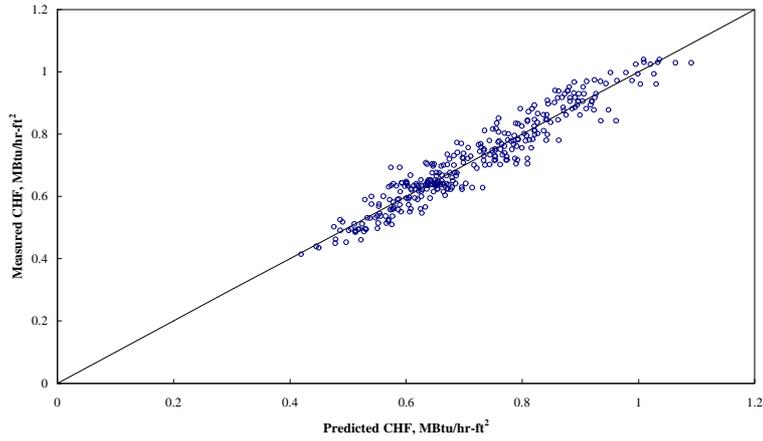
3. PLUS7



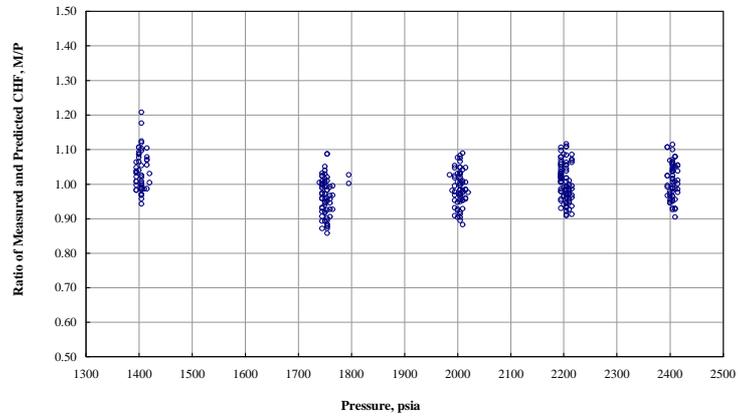
4. (typical test section)



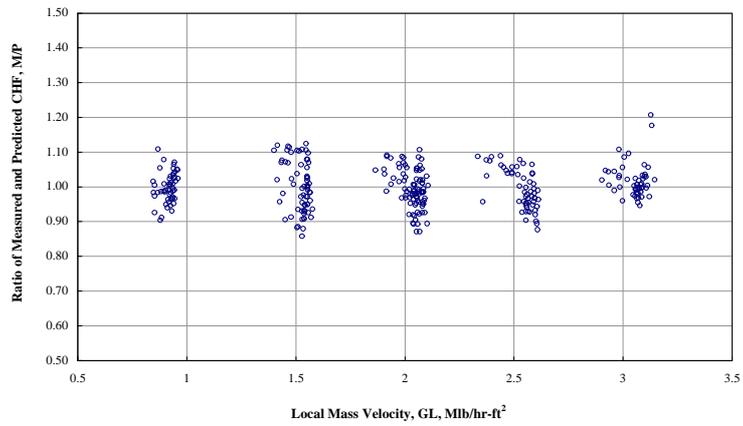
5. (thimble test section)



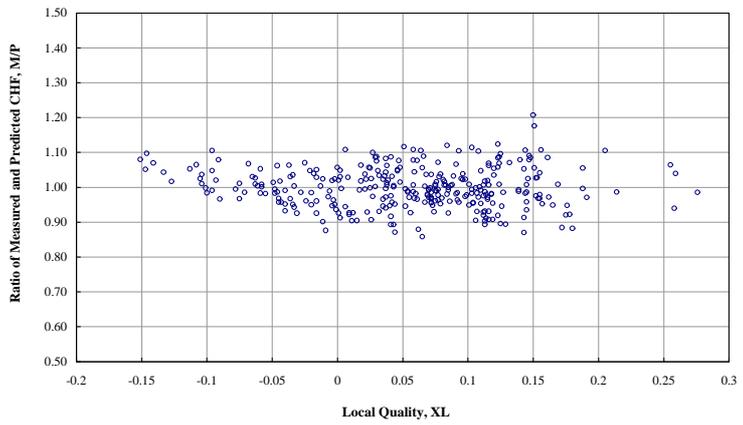
6. KCE-1



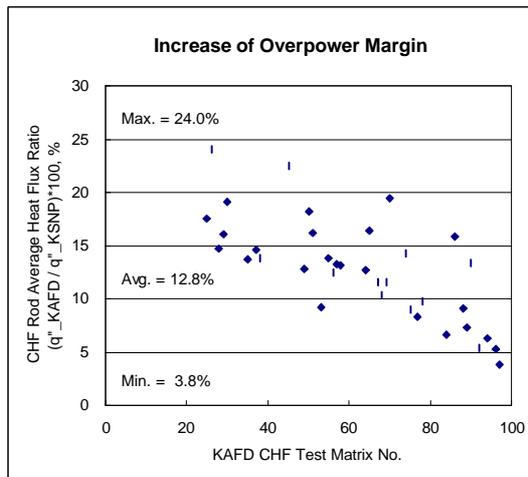
7. KCE-1



8. KCE-1



9. KCE-1



10. PLUS7