

CFD Analysis for the Development of Next Generation Fuel Mixing Vane

493

가
가

FLUENT5.5

. CFD

2

RNG κ - ϵ

가

가

Abstract

Some models of flow mixing vane were selected to evaluate the flow characteristics in subchannel for the development of mixing vane of PWR Next Generation fuel assembly. The analyses of those mixing vane models were done through a numerical investigation using the computational fluid dynamics code, FLUENT5.5. The mixing vane models were determined maintaining the magnitude of mixing vane constantly based on the current mixing vanes. CFD analysis was done for two subchannels including the space grid and mixing vane. The RNG κ - ϵ turbulent model is used. As a result of evaluation, the models that have the flow characteristic similar to the current mixing vane making swirl flow in subchannel and crossflow between subchannels, have advantage in pressure drop. But hot spots on surface of fuel rod still remain locally. The others show the effective flow mixing performance and the reduced the azimuthal temperature gradient of fuel rod, but disadvantage in pressure drop.

1.

(Mixing Vane)

2

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가

가

. Karoutas ⁽¹⁾

가

3

CFD

CFDS-FLOW3D

, ⁽²⁾

CFD

CFX-4

가

가

AOA(Axial Offset Anomaly)

가

가

가

가

가

CFD

가

가 CFD

FLUENT5.5^[3]

2.

2.1

가

, 8
 . 1 가
 . 2
 가 . 가
 A ,
 가 . B ~ F
 2 4 ,
 ,
 가
 가 가 가 . G
 가 가 가
 90° Cone Flow
 H G
 , 가 . C G
 가 . 1 A
 . 1 .

2.2

가 가 ,
 CFD FLUENT^[3] . CFD
 1 11 ,
 2 .
 가 ,
 3" 10.5" . 3"
 10.5" IFM(Intermediate Flow Mixer)
 . 가
 0.018"
 (Non-structured Grid) , 1×10^6
 가
 No-Slip SIMPLE Algorithm, RNG κ - ϵ
 10^{-4} 가
 2000 . 가

3 , 5.18 m/s(17 ft/sec),
 594.26 K(610), 4% .
 945000 W/m²(300000 Btu/hr-ft²) 가 .

2.3 가

3 4 . 3
 가 4
 1/3 (6.67 ") 1/2 (10") .
 4 4
 . A C 가 ,
 1/3 가
 . A 1/2 , C 1/2
 가 , B, D F
 . B 1/3 . D F B
 1/3 1/2
 . E
 . G
 H 1/3
 . 5 . G H
 . 가 AOA
 가
 . 6
 1/2 ,
 . 5 G H
 가
 7 A
 . 2
 . G A 가

3.

가 ,

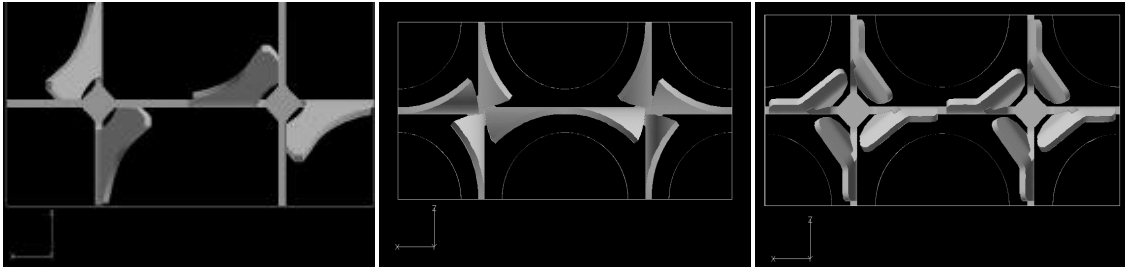
가

4.

1. Karoutas Z., Gu C.Y. and Scholin B., 1995, "3-D flow analyses for design of nuclear fuel spacer," Proc. Of the 7th Int. Meeting on Nuclear Reactor Thermal-Hydraulics, New York, United State, September 10-15.
2. 2 , " , "2001 ,
3. FLUENT User's Manual, Fluent Inc., 2001

1.

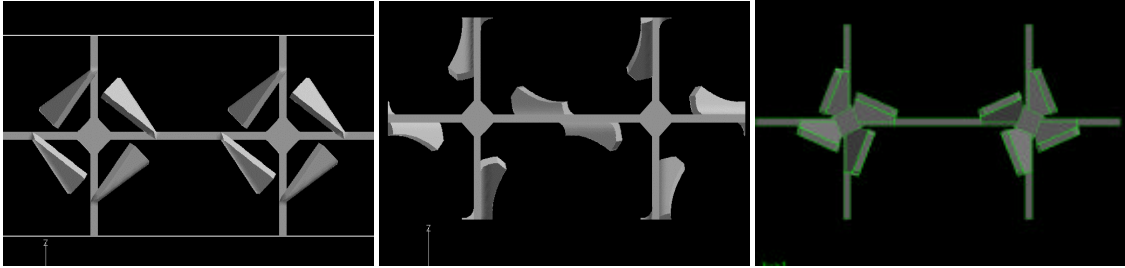
| | A | B | C | C-1 | C-2 | D | E | F | G | G-1 | H |
|------|------|------|------|------|------|------|------|------|------|------|------|
| | 1 | 0.96 | 1.34 | 1.07 | 0.93 | 1 | 1.02 | 0.74 | 1 | 0.5 | 1 |
| (in) | 1.79 | 2.09 | 1.89 | 1.89 | 1.89 | 2.09 | 2.09 | 1.79 | 1.79 | 1.79 | 1.79 |



A

B

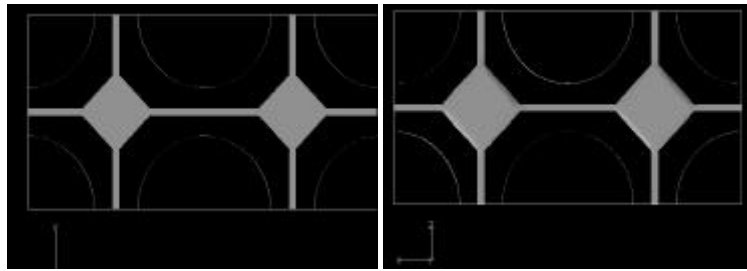
C



D

E

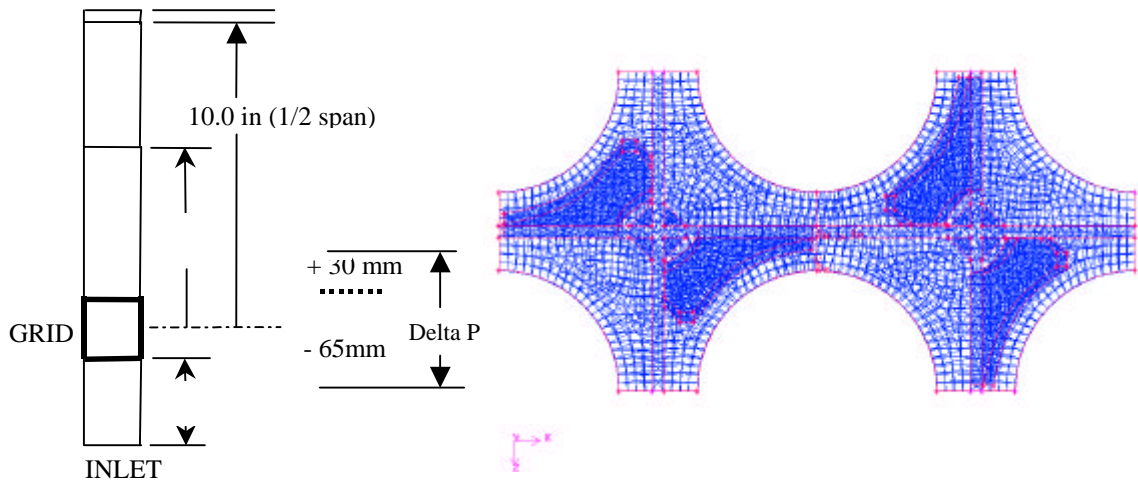
F



G

H

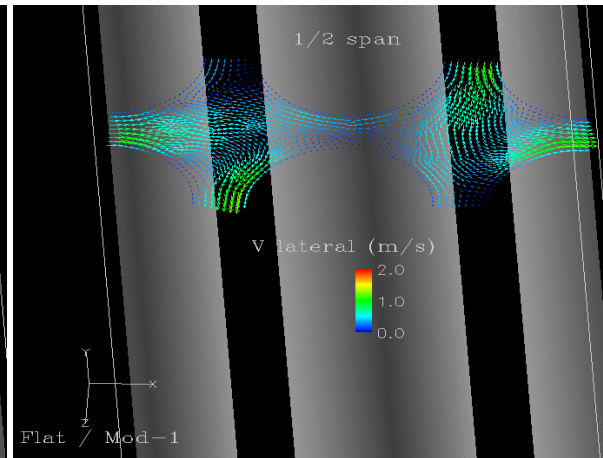
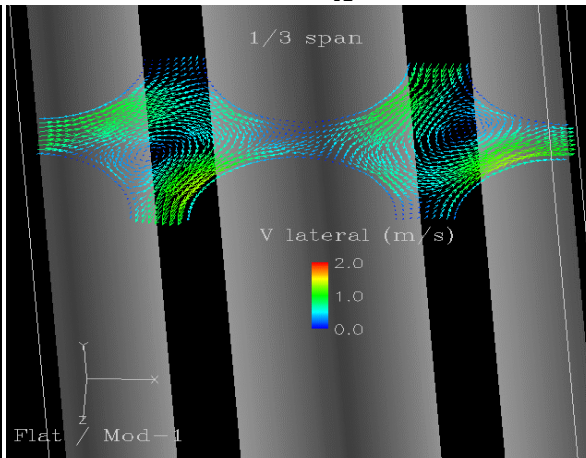
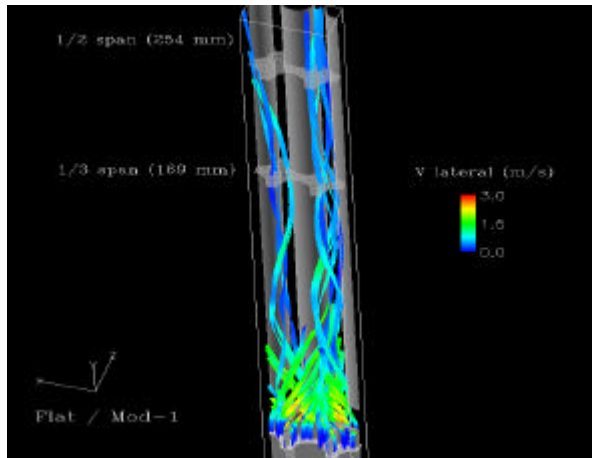
1.



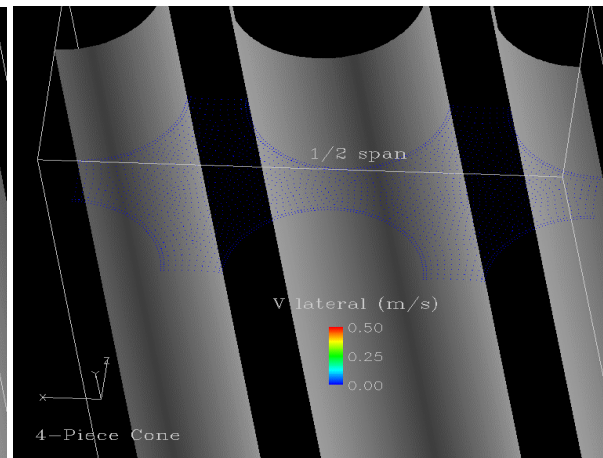
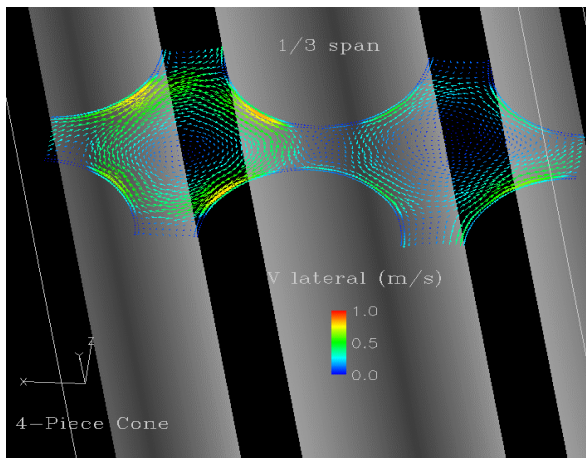
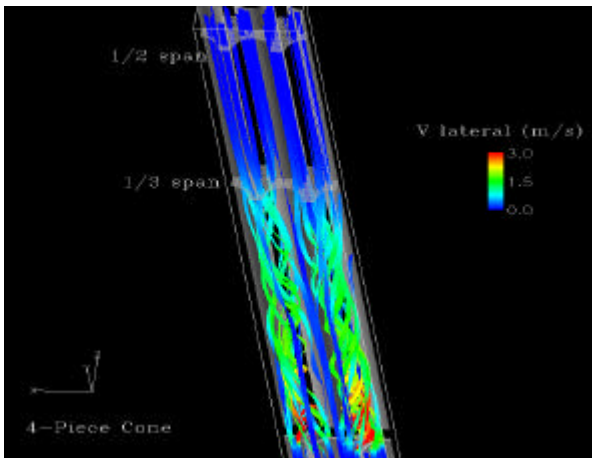
2.

()

A



B



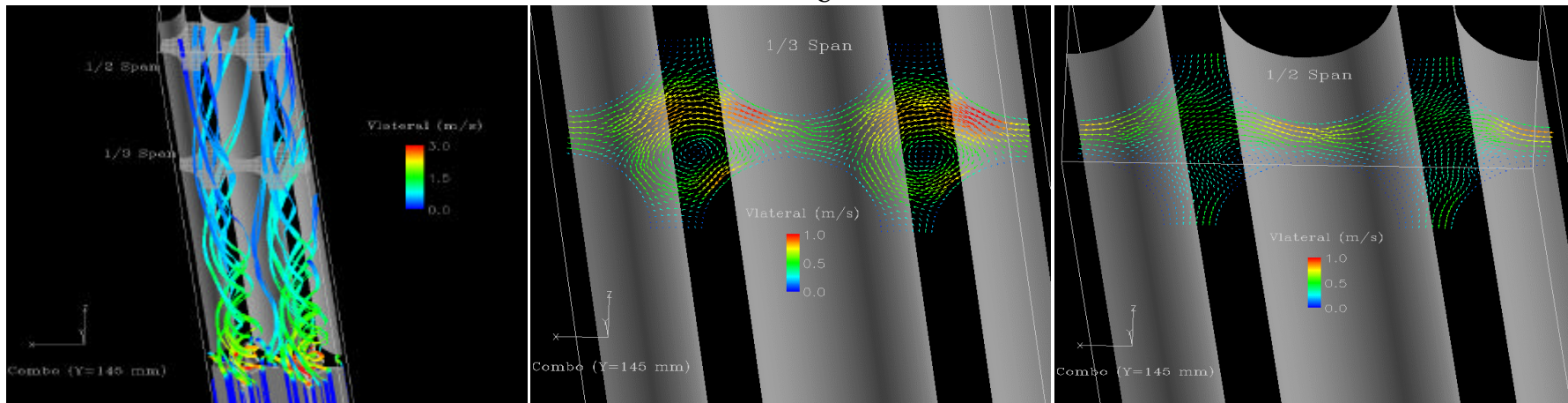
6.67"

10"

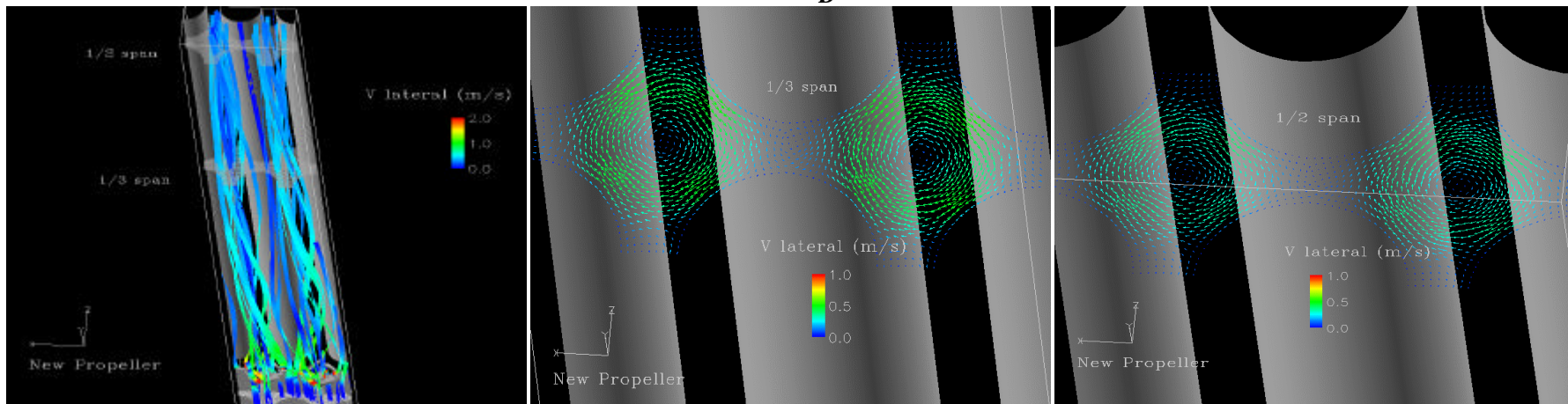
3. CFD

(1/2)

C



D



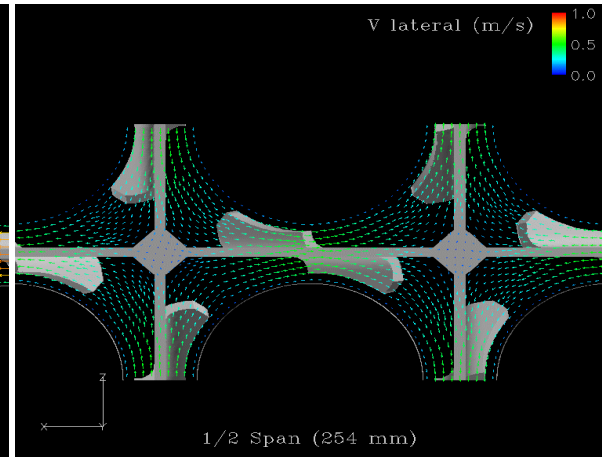
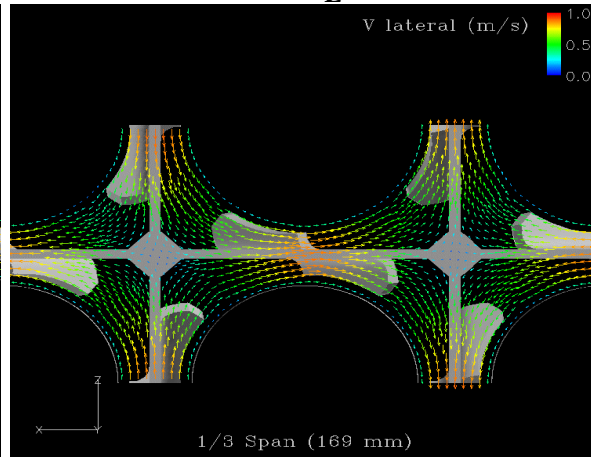
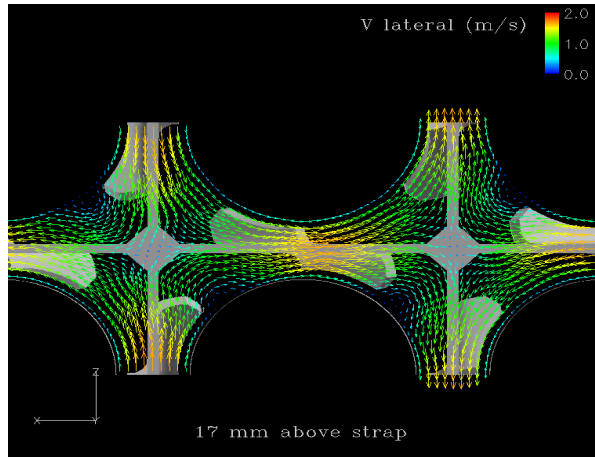
6.67"

10"

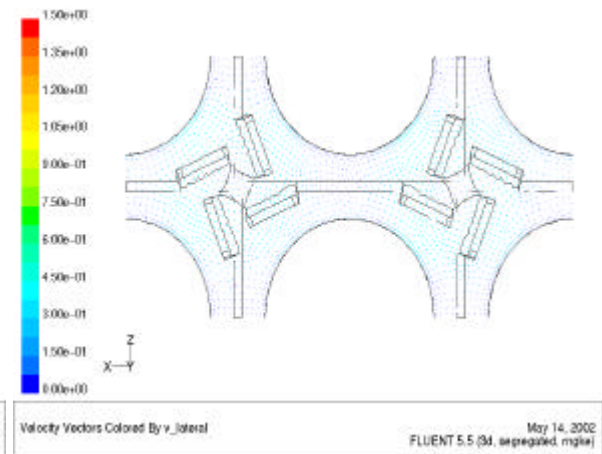
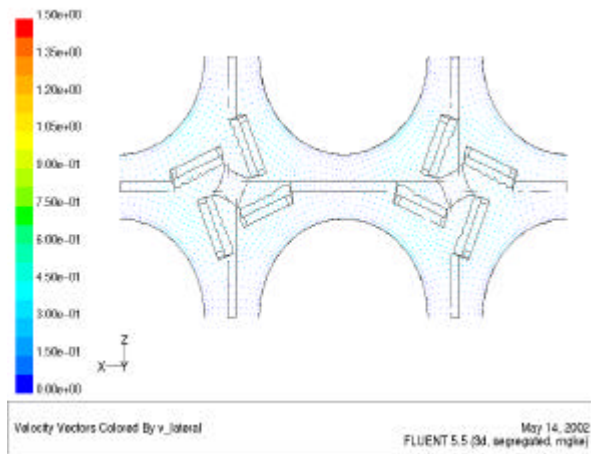
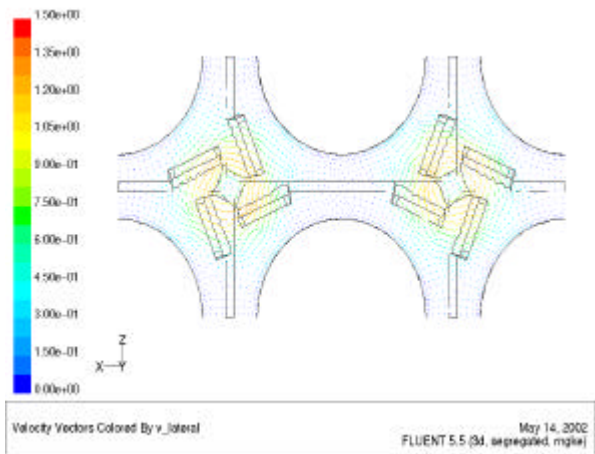
3. CFD

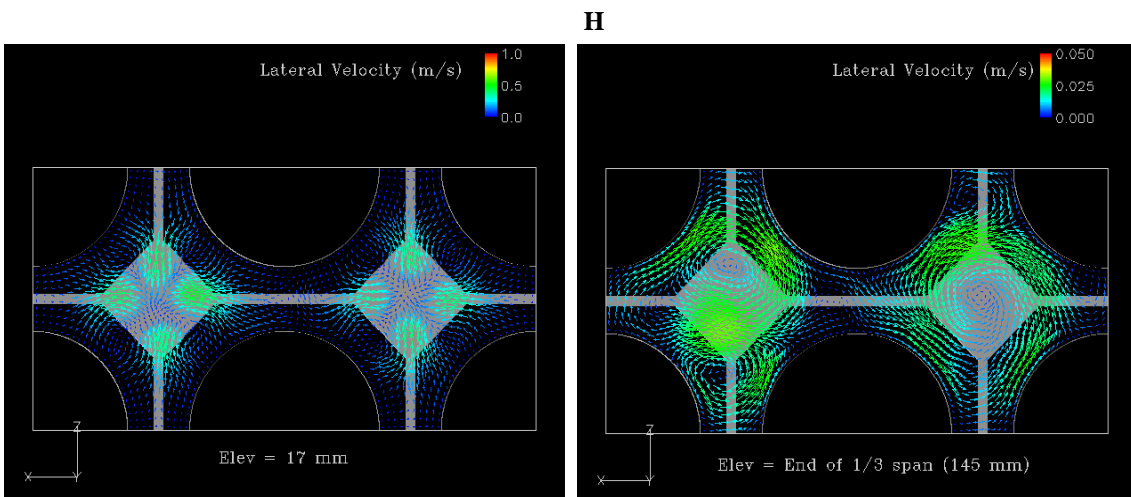
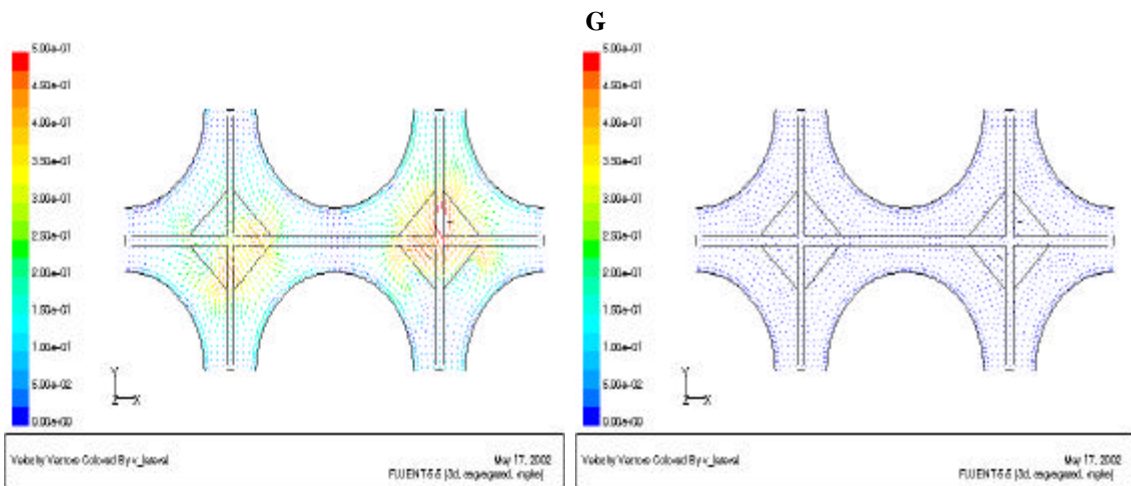
(2/2)

E



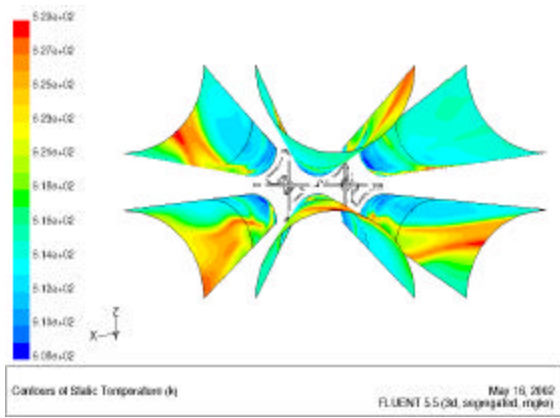
F



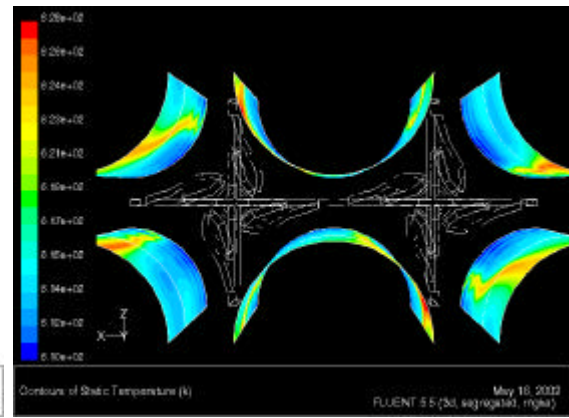


4.

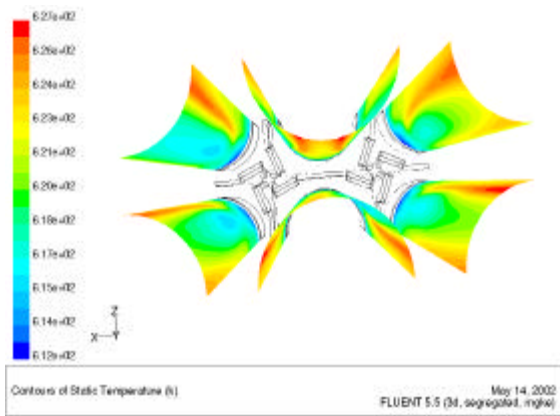
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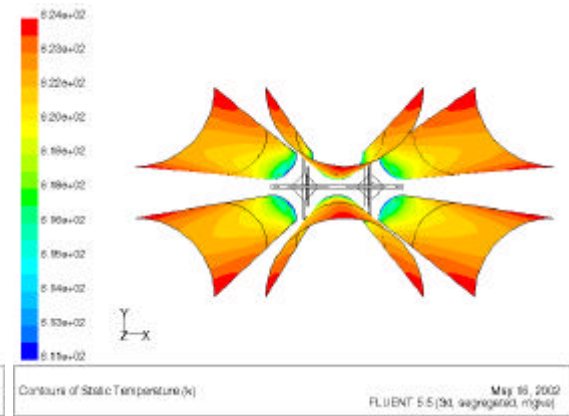
A



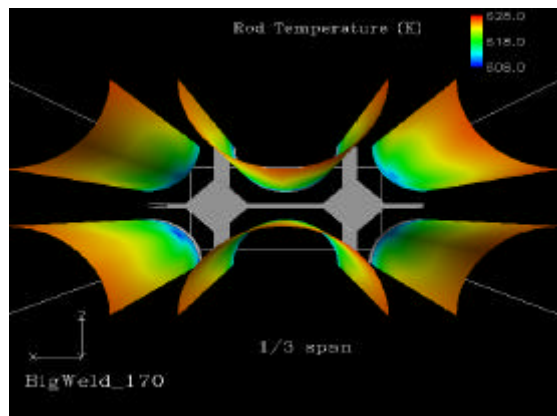
C-1



F

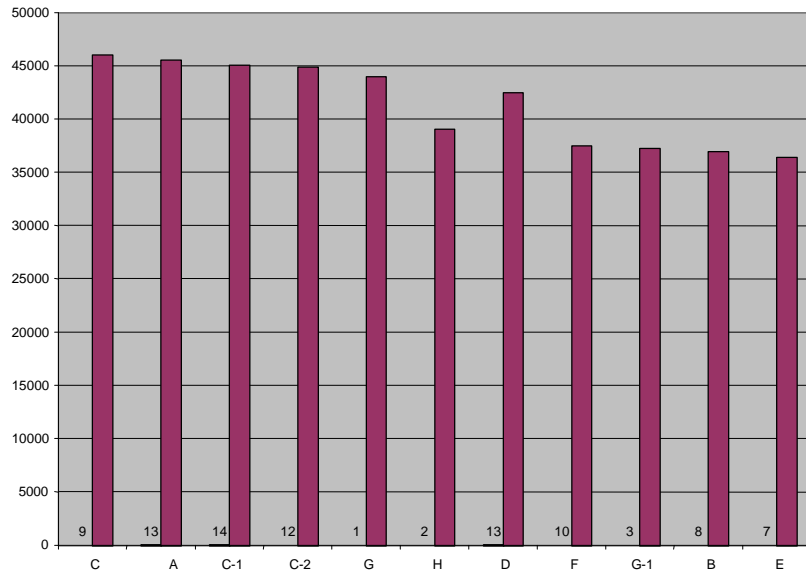


G-1

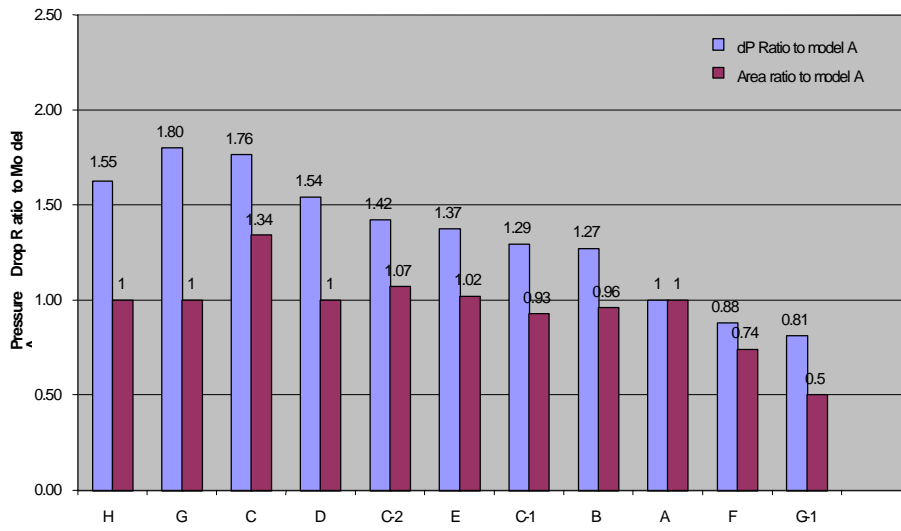


H

5.



6. 10"



7. A