

PSA

Standardization of Domestic Human Reliability Analysis and Experience of Human Reliability Analysis in Probabilistic Safety Assessment for NPPs under Design

가

가

가

가

가

가

Abstract

This paper introduces the background and development activities of domestic standardization of procedure and method for human reliability analysis (HRA) to avoid the intervention of subjectivity by HRA analyst in probabilistic safety assessment (PSA) as possible, and the review of the HRA results for domestic nuclear power plants under design studied by Korea Atomic Energy Research Institute. We identify the HRA methods used for PSA for domestic NPPs and discuss the subjectivity of HRA analyst shown in performing a HRA. Also, we introduce the PSA guidelines published in USA and review the HRA results based on them. We propose the system of a standard procedure and method for HRA to be developed.

1.

가

가

(risk-informed)

가(Probabilistic

Safety Assessment: PSA)

ASME PRA Standard(ASME

PRA) [1] NEI PRA Peer Review Guideline(

NEI PRA) [2]

PSA

PSA (generic)

가

(plant specific)

PSA

PSA

PSA

PSA

가

가

(Human Reliability Analysis: HRA)

PSA

가

HRA

PSA

5

HRA

HRA

가

HRA

HRA

PSA

가

PSA

HRA

가

HRA

2002 8

HRA

가

가

[3].

HRA

PSA

HRA

가

가

PSA

가

HRA

[3]:

●

HRA

가

●

(ASME NEI PRA)

HRA

●

HRA

가

HRA

PSA HRA . 2 PSA
HRA , HRA ASEP(accident sequence evaluation program)[4] THERP(technique for human error rate prediction)[5]
. 3 ASME NEI PRA ,
HRA . 4 가 HRA
ASME NEI PRA
HRA

2. HRA ASEP/THERP

PSA HRA HRA
ASEP [4] THERP[5]

SHARP(systematic human action reliability procedure)[6]

PSA
HCR [7], THERP [5], ASEP [4], SLIM [8], cause based decision tree [9]
CREAM [10], ATHENA [11]

PSA

1. HRA

No	PSA	PSA			
1	3,4 1,2	PSA	KOPEC	1992	HCR/THERP
2	3,4	PSA	KAERI	1993	ASEP/THERP
3	2,3,4	PSA	KAERI	1997	ASEP/THERP
4	3,4	PSA	KAERI	1997	ASEP/THERP
5	KNGR	PSA	KOPEC	1999	THERP
6	5,6	PSA	KAERI	2001	ASEP/THERP
7	5,6 (/)	PSA	KAERI	2001	THERP
8	5,6	PSA	KEPRI	2002	ASEP/THERP
9	1	PSA	ACT		ASEP/THERP
10	2	PSA	KOPEC		ASEP/THERP
11	1	PSA	KOPEC		ASEP/THERP
12	3,4 1,2 (update)	PSA	KOPEC		HCR/THERP (가 ,)

KOPEC: , KAERI: , ACT: -

1 HRA 1992 HRA [3].
 HRA 1992 3,4 1,2 PSA
 . PSA HRA 3,4 1,2
 ASEP THERP .

PSA
 2가 ((pre-accident) (post-accident)) 3가

[6]:

- A ():
- B :
- C ():

ASEP THERP

A 가 . ,
 C 가 . B
 . 1
 PSA ASEP

THERP

(decomposition)

가 ,

- A : ((omission error) + (commission error))
- C : (diagnosis error) + (execution error)

A

(omission)

(commission error)

A

PSA

가(quantitative risk

assessment: QRA),

A

THERP ASEP

PSA

A

가

THERP

ASEP

A

1

PSA

, A 가

가

A

C

가

ASEP

THERP

1 PSA C 가 C HRA , , C

[4, 5]. 가

3,4 1,2 PSA C 가 ASEP
 THERP HRA 가
 ASEP THERP C 가 :

- = () X (factors)
- = Σ [X]

가

, , / , .

가

(feedback) . HRA 가
 (performance shaping factors: PSF)

가 “X” 가
 A HRA “ (reluctance) B HRA
 . A “X” (dynamic action) 가 B
 (step-by-step action) 가 HRA

가 . ASEP THERP
 가 .

-
-
- (, ,)
- 가
-
-
- 가

-
-
-
-

3. ASME NEI PRA HRA

ASME NEI PRA HRA

ASME PRA 6 :

- : PRA , ,
- : ,
- 가 : PRA , Capability I, II, III
- 가 : , (HLR: high level requirements)
(SR: supporting requirements), 9 (, , ,)
- PRA : PRA
- : PRA , NEI PRA

PRA Capability II

2. ASME PRA

	Capability Category I	Capability Category II	Capability Category III
		, , (SSC)	
	/		

NEI 4 3 :

- : , ,
- PRA : , , ,
- : , , 11 PRA
(, 가, , , , ,) ,
-

NEI PRA 3 4가 3
 Grade 3
 PRA ASME Capability II NEI grade 3
 가
 Capability II
 Grade 3

3. NEI PRA Grade

	Grade 1	Grade 2	Grade 3	Grade 4
	-NRC Generic Letter 88-20 PRA grade 1	- Grade 2 (risk ranking) Grade 2 Grade 1	- Grade 3 PRA 가 가 - Grade 3 Grade 2	- 가 - Grade 4 Grade 1, 2, 3
	IPE for GL 88-20 requirement Assessing severe accident vulnerabilities	MOV ranking, Maintenance rule support	Graded QA/IST/ISI/on-line maintenance, single TS changes	-Replace technical specification. with an on-line risk monitor

GL: generic letter, MOV: motor operated valve, QA: quality assurance, IST: in-service testing, ISI: in-service inspection, TS: technical specification

4. ASME NEI PRA

	ASME 9 HLR 34 (SR)	NEI - 30
4 HLR - 15	-3 -2 PSA -3 / 가 -7	-3 -4 (, - 18 (, ,
4 HLR - 18	-4 PSA -2 가- 9 -3) 가 -2 -3
1 HLR - 1		

4 ASME PRA 9
 (HLR) HLR 34 (supporting requirements: SR)
 HLR HLR

PSA , 가 . NEI PRA
30 , ASME PRA

가 . 2
:

- , PSA
- 가
-
- ASME, NEI PRA PSA HRA
- 2 가
- PSA HRA ,
HRA 가

4. HRA HRA

가 HRA ASME PRA 가
HRA

5 가 HRA [12]. ASME
PRA 가 ,
Capability I 가 , NEI
Grade II 가 PSA
ASME capability

II NEI grade 3

5. HRA

No	PSA		
1	3,4	1993	ASEP IPE HRA
2	2,3,4	1997	ASEP IPE HRA
3	3,4	1997	ASEP IPE HRA
4	5,6	2001	ASEP IPE HRA
5	5,6 (/)	2001	THERP IPE HRA

*IPE: individual plant examination

6. 3,4 PSA HRA

1	/	& ,
2	()	.
3	RPS/ESFAS 가	HRA
4	가	1
5	가	
6		가 , HRA , 가
7	(AOP)	,
8		,
9	, (time window),	(cues) (time window) (:)
10	(PSFs)(:)	HRA
11		HRA
12	가	HRA
13	(credit)	HRA
14		HRA
15	, PSA	HRA

PSA

HRA

6

3&4

HRA

가

PSA HRA

●

가

●

(AOP)

●

(time analysis)

(cues)

●

/

●

가(

,

,

-)
- HRA ASME NEI PRA
- HRA 가 HRA :
- :
- : A
- (CCF)가 , 가 , C
- 가
- : 가.
- ASEP (ASME PRA)
- : 가.
- THERP (ASME PRA)
- 가 - ,
- 가 -
- PSA (, , /)
- : NEI PRA

5.

HRA HRA HRA ASME

NEI HRA HRA HRA

HRA 1992 3,4 1,2 PSA

ASEP THERP HRA

가 (, ,),

HRA

ASME PRA Capability I NEI PRA

Grade 2

HRA , HRA
HRA
PSA
HRA
HRA 가

- [1]. ASME, "Standard for PRA for NPP Applications" , Rev. 14A, 2001
- [2]. NEI, "PRA Peer Review Process Guidance" , NEI-00-02, Rev. A3, 2000
- [3]. , 1 HRA 가 : , 2002,
- [4] A.D.Swain, "Accident Sequence Evaluation Program Human Reliability Analysis Procedure", NUREG/CR-4772, S.N.L, Feb. 1987
- [5]A.D.Swain and H.E.Guttman, "Handbook of Human Reliability Analysis with Emphasis on Nuclear Power Plant Applications", NUREG/CR-1278, S.N.L, Aug. 1983
- [6] G.W, Hanaman et al., "Systematic Human Action Reliability Procedures(SHARP)", EPRI/NP-3583, 1984
- [7] G.W.Hanaman et al., "Human Cognitive Reliability Model for PRA Analysis" , NUS-4531, NUS Corporation, Dec. 1984
- [8] Embrey, D., "SLIM-MAUD: An Approach to Assessing Human Error Probabilities Using Structured Expert Judgment", NUREG/CR-3518, 1984
- [9].G.W. Parry et al., "An Approach to the Analysis of Operator Actions in Probabilistic Risk Assessment", EPRI/TR-100259, 1992
- [10]. Hollnagel, E., "Cognitive Reliability and Error Analysis Methodology", Elsevier, 1998
- [11] "Technical Basis and Implementation Guidelines for a Technique for Human Event Analysis(ATHENA)", NUREG/CR-1624, 2000
- [12] , 1 HRA 가 : HRA -