

가 BNCT
A Study on Epithermal Neutron Beam Quality Improvement
for Accelerator-Based BNCT

17

BNCT
(broad) 가
() ,
, 가
(narrow)
. 가 PLINS , AlF₃
Al-27, Ni-60 가

ABSTRACT

Up to now, all epithermal neutron beam designs for BNCT have been focused on generating neutron beams that have broad spectra covering all epithermal energy regions. It is reasonable to generate epithermal neutron beam for BNCT. However, when we consider a tumor which is locally seated in brain, it is desirable to use a locate-specific epithermal neutron beam which has a narrow peak at an epithermal energy region. From this idea, it was investigated the possibility of generating a narrowly peaked epithermal neutron beam using accelerator. Neutron source is calculated with PLINS code. It is found that epithermal neutron beams, those have narrower band spectra than previously designed beam, can be generated with properly combined

moderators; AlF₃, Al-27, and Ni-60.

1.

(BNCT, Boron Neutron Capture Therapy) ,
 , 2
 , BNCT
 가 가 가
 가 ,
 (GBM,
 Glioblastoma Multiforme)
 LET(Linear Energy
 Transfer)
 BNCT (4eV E_n 40keV) [1].
 (intensity) (quality)
 BNCT
 (flux) 1×10^9 neutrons/cm²-sec 1
 가
 (forward directional property)
 가 가 가
 BNCT 가 ,
 가
 (beam shaping assembly)
 BNCT
 (broad)
 ()

, 가

(narrow)

2.

2.1

가

BNCT

Li-7 Be-9

Li-7

가

Li-7

, ${}^7\text{Li}(p,n){}^7\text{Be}$

PLINS(Proton Lithium Induced Neutron Source)

[2]

2.5 MeV

1

PLINS

2.5 MeV

(,)

0.8 MeV

가

가

2.2

2.2.1

가

-

(Σ_s)

-

(ξ in lethargy)

-

(Σ_a)

,

(Z)

, ξ

(radiative capture)

가

가

MCNP

(resonance scattering cross section)

가

He F

가

F

(Z)

F

($\Sigma\phi$)

2.2.2 MCNP

MCNP

15 cm

30 cm Pb

가

0.8

MeV

가

MCNP

1 cm

20 cm

25 cm

5 cm, 10 cm

15 cm

MCNP4C

ENDF-VI

2.3

가

가

IAEA

[3]

BNCT

FWHM(Full Width at

Half Maximum)

가

가

2.3.1

(> 40 keV)

LET

가

$D_n < 2.0 \times 10^{-13} \text{Gy-}$

$\text{cm}^2/n_{\text{epi}}$

2.3.2

$$f_{\text{thermal}} / f_{\text{epihermal}} < 0.05$$

2.3.3

BNCT 가
가
(n,γ)
 $D_{\gamma} < 2.0 \times 10^{-13} \text{ Gy-cm}^2 / n_{\text{epi}}$

2.3.4

가

$$J_{\text{total}} / f_{\text{total}} > 0.7$$

2.3.5

가 FWHM

3.

3.1

PLINS BNCT
MCNP 가

MCNP

LiF, AlF₃, CrF₂, S₂F₁₀, TiF₃, FeF₂

AlF₃

Al-27 Ni-60

2 3

4 ~ 6

가

3.2

가

가

1

2

IAEA

가

0.60

20 cm

Al-27

10 cm

가

2

(peak) 가

FWHM

lethargy

FWHM

2

2.1

가

Ni-60

0.4 ~ 0.6, Al-27

0.7 ~

1.2

가

4.

, 가

가

BNCT

가

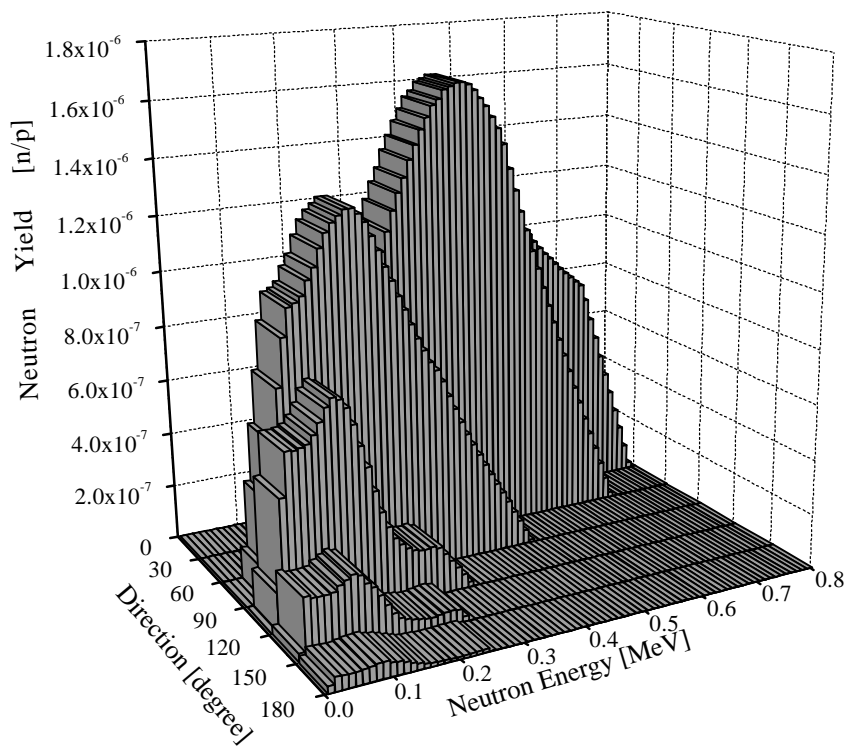
가

가

VOXEL

가

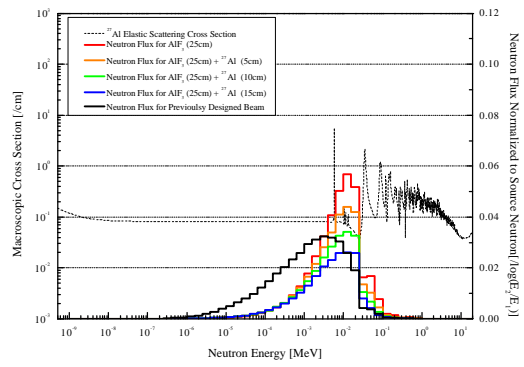
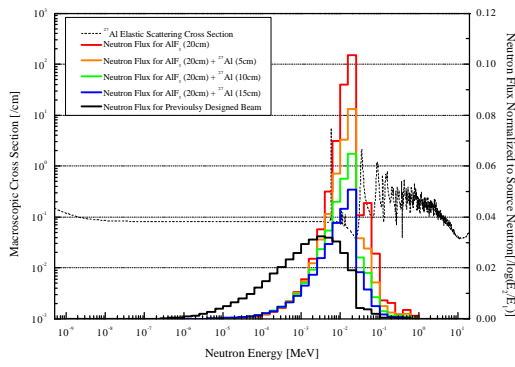
1. Mi Young PAEK, Chi Young HAN, and Jong Kyung KIM, "A Characteristic Analysis of a Accelerator-Based Epithermal Neutron Beam for Boron Neutron Capture Therapy," ISORD-1, 2001.
2. , "가 BNCT ," , 1998.
3. IAEA, "Current Status of Neutron Capture Therapy," IAEA-TECHDOC-1223, May, 2001.



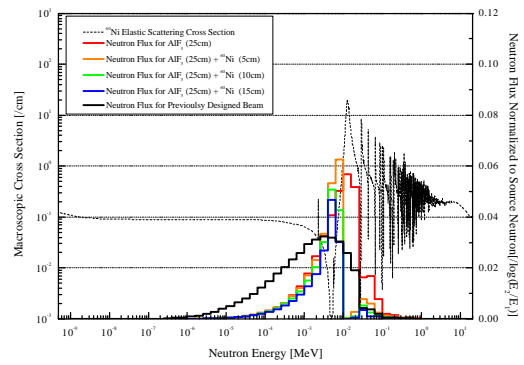
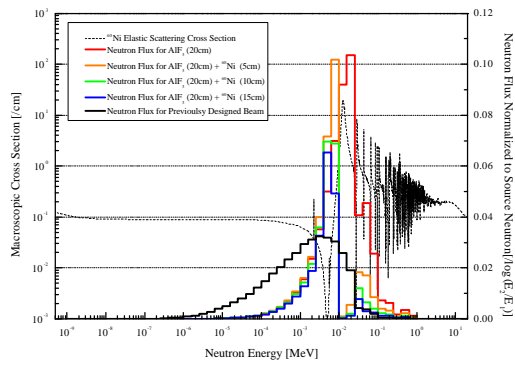
1. PLINS

${}^7\text{Li}(n){}^7\text{Be}$

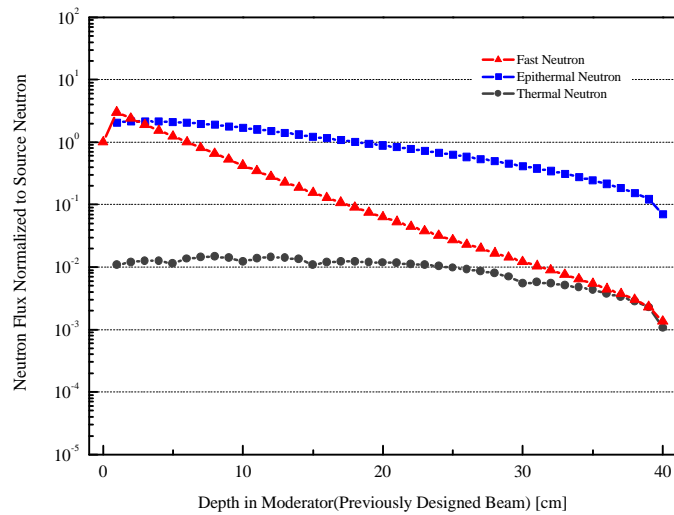
($E_p = 2.5$ MeV)



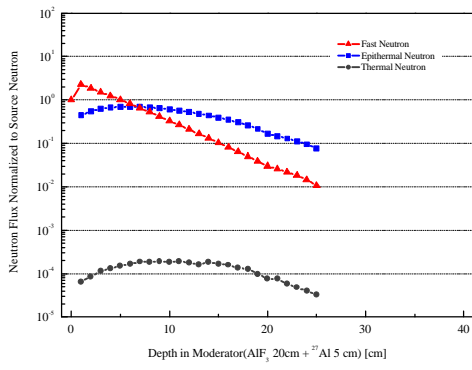
2. ($\text{AlF}_3 + ^{27}\text{Al}$)



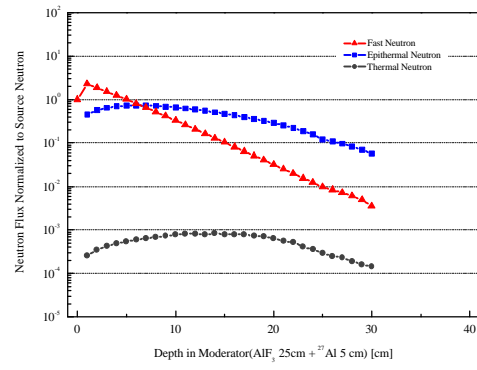
3. ($\text{AlF}_3 + ^{60}\text{Ni}$)



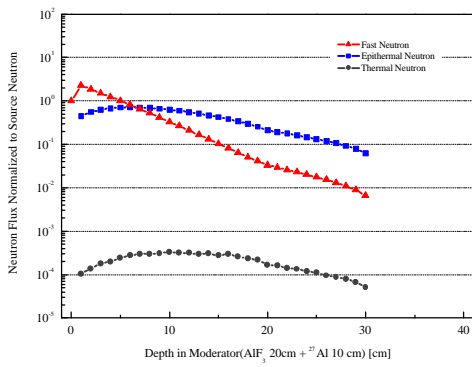
4.



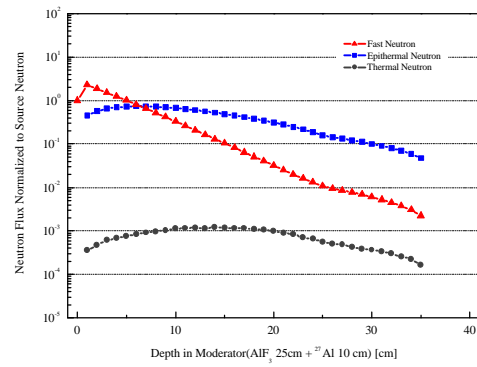
(1) AlF_3 20cm + ^{27}Al 5cm



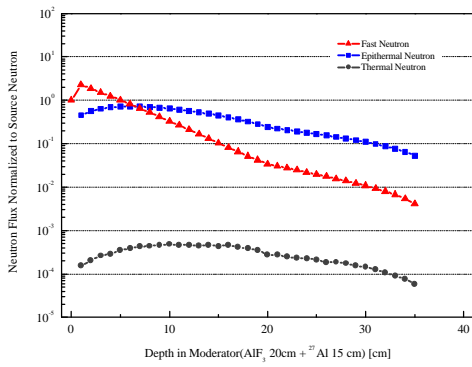
(4) AlF_3 25cm + ^{27}Al 5cm



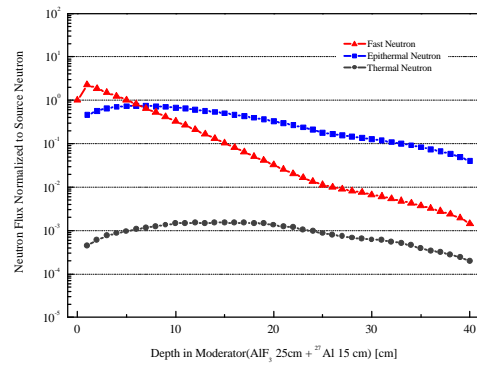
(2) AlF_3 20cm + ^{27}Al 10cm



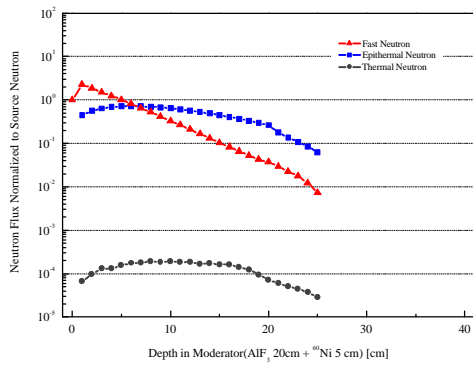
(5) AlF_3 25cm + ^{27}Al 10cm



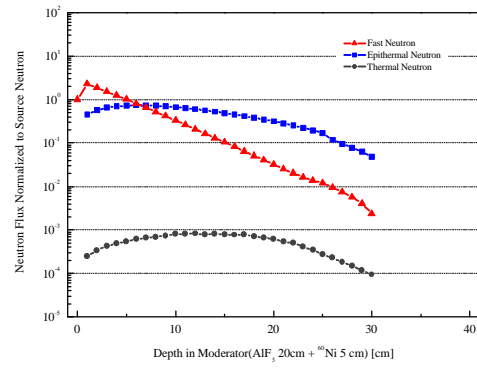
(3) AlF_3 20cm + ^{27}Al 15cm



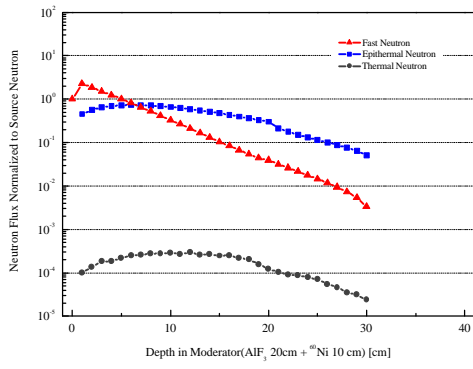
(6) AlF_3 25cm + ^{27}Al 15cm



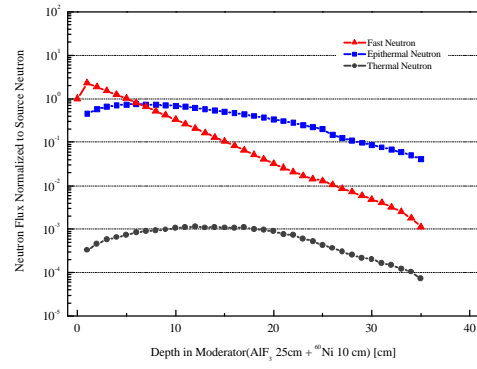
(1) AlF₃ 20cm+⁶⁰Ni 5cm



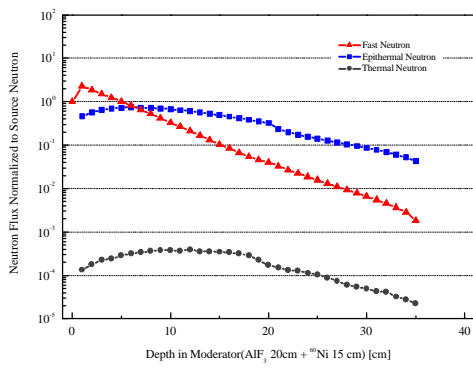
(4) AlF₃ 25cm+⁶⁰Ni 5cm



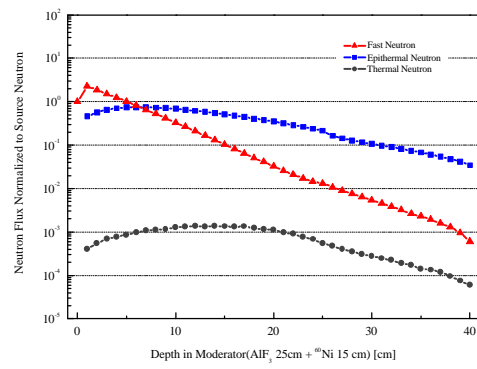
(2) AlF₃ 20cm+⁶⁰Ni 10cm



(5) AlF₃ 25cm+⁶⁰Ni 10cm



(3) AlF₃ 20cm+⁶⁰Ni 15cm



(6) AlF₃ 25cm+⁶⁰Ni 15cm

1.

(1)

Moderator	Forward Directional Property, (J/f)	Fast Neutron Dose, D_n	Thermal Flux/ n_{epi} , (f_{th} / f_{epi})	Gamma Dose, D_γ
Previously Designed Beam	0.59	3.76261E-14	0.0035	7.50583E-15
AlF ₃ 20 cm + ²⁷ Al 5 cm	0.61	3.29665E-13	0.0002	1.93781E-14
AlF ₃ 20 cm + ²⁷ Al 10 cm	0.62	2.58783E-13	0.0004	1.46778E-14
AlF ₃ 20 cm + ²⁷ Al 15 cm	0.62	2.08231E-13	0.0006	1.28650E-14
AlF ₃ 25 cm + ²⁷ Al 5 cm	0.61	1.46543E-13	0.0012	1.31019E-14
AlF ₃ 25 cm + ²⁷ Al 10 cm	0.62	1.19979E-13	0.0017	1.16371E-14
AlF ₃ 25 cm + ²⁷ Al 15 cm	0.62	1.00759E-13	0.0025	1.11472E-14
AlF ₃ 20 cm + ⁶⁰ Ni 5 cm	0.60	1.05355E-13	0.0001	1.55972E-14
AlF ₃ 20 cm + ⁶⁰ Ni 10 cm	0.62	6.43569E-14	0.0001	1.23313E-14
AlF ₃ 20 cm + ⁶⁰ Ni 15 cm	0.63	4.47608E-14	0.0001	1.09982E-14
AlF ₃ 25 cm + ⁶⁰ Ni 5 cm	0.60	4.21816E-14	0.0004	1.41886E-14
AlF ₃ 25 cm + ⁶⁰ Ni 10 cm	0.62	2.71136E-14	0.0004	1.26138E-14
AlF ₃ 25 cm + ⁶⁰ Ni 15 cm	0.63	1.89086E-14	0.0003	1.09663E-14

2.

(2)

Moderator	Neutron Flux Normalized to Source Neutron			Neutron Mean Energy [keV]	FWHM
	Thermal Neutron	Epithermal Neutron	Fast Neutron		
Previously Designed Beam	1.06177E-03	7.09851E-02	1.35661E-03	5.636	2.1
AlF ₃ 20 cm + ²⁷ Al 5 cm	3.32470E-05	7.62110E-02	1.06791E-02	22.881	0.7
AlF ₃ 20 cm + ²⁷ Al 10 cm	5.11210E-05	6.32601E-02	6.58002E-03	19.154	0.7
AlF ₃ 20 cm + ²⁷ Al 15 cm	5.91428E-05	5.26252E-02	4.08735E-03	16.377	0.8
AlF ₃ 25 cm + ²⁷ Al 5 cm	1.43174E-04	5.67272E-02	3.58261E-03	13.938	1.1
AlF ₃ 25 cm + ²⁷ Al 10 cm	1.65056E-04	4.75856E-02	2.25104E-03	12.028	1.2
AlF ₃ 25 cm + ²⁷ Al 15 cm	2.00985E-04	3.98790E-02	1.42889E-03	10.617	1.2
AlF ₃ 20 cm + ⁶⁰ Ni 5 cm	2.88178E-05	6.20402E-02	7.26700E-03	18.890	0.4
AlF ₃ 20 cm + ⁶⁰ Ni 10 cm	2.41136E-05	5.07508E-02	3.32476E-03	14.258	0.5
AlF ₃ 20 cm + ⁶⁰ Ni 15 cm	2.26378E-05	4.28522E-02	1.84471E-03	11.414	0.5
AlF ₃ 25 cm + ⁶⁰ Ni 5 cm	9.48093E-05	4.85234E-02	2.33081E-03	10.298	0.5
AlF ₃ 25 cm + ⁶⁰ Ni 10 cm	7.40473E-05	4.08117E-02	1.13258E-03	8.084	0.6
AlF ₃ 25 cm + ⁶⁰ Ni 15 cm	6.00655E-05	3.44552E-02	6.06204E-04	6.810	0.5