

Thermal Size Evaluation for Double Wall Tube Steam Generator in Sodium Cooled Liquid Metal Reactor

150

가

가

가

KALIMER

2

70%

Abstract

The results obtained in assessing the thermal sizing of double wall tube steam generator concept are presented in this paper. The concept provides double barrier between the sodium and the steam. This will result in improved reliability of SG compared to the KALIMER design using single wall tubes. The condition between the inner and outer tubes affects the heat transfer from sodium to steam. To estimate heat transfer performance of the concept, the required heat transfer area was calculated, and compared with KALIMER SG. The required heat transfer area for DWTSG is about 2 times larger than that of KALIMER 150 SG. In the case of eliminating IHTS, the area is decreased to the 70% of the total area of intermediate heat exchanger and steam generator.

1.

가

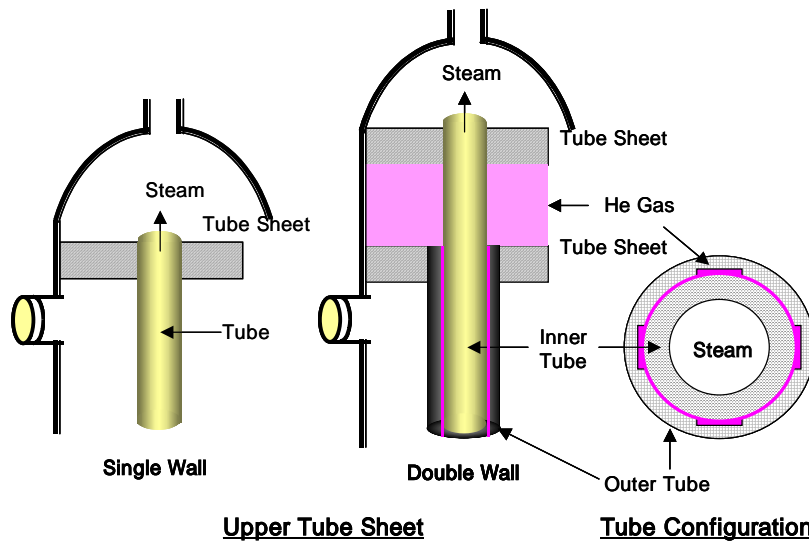
1,2,3
 SG / (Pb-
 Bi) 가
 Pb Pb-Bi 가 가 ,
 가 ,

(Double Wall Tube Steam Generator :DWTSG) 30

Bi , Pb Pb-
 EBR-II 1964
 30 SG SWR
 4 ,
 가 DWTSG
 가 , DWTSG
 /

2. DWTSG

가 (duplex-tube arrangement) , 가 He
 He
 1 DWTSG 2
 He 가
 가 (pre-stressed)
 가 (0.2 mm) ,



1

3.

3.1. DWTSG

DWTSG KALIMER-150 SG

가 , SG

. DWTSG 1 . KALIMER-150 SG

EBR-I⁵ Forster

Wheeler Energy⁶

5.3mm KALIMER 50% 가 .

2

DWTSG

2

1

Parameter	Value
/ , mm	16 / 21.2
/ , mm	21.216/ 26.67
He ,mm	0.008
/ , mm	2.6 / 2.7
, mm	52
	2-1/4Cr-1Mo
	He

2 KALIMER-150

			/
T _{hot} , °C	529.8	511	230
T _{cold} , °C	385	339	483.2
, kg/s	1071.2 *	901.8	87.725

3.2.

$$Roy^7 = \frac{28,400W/m \cdot ^\circ C}{St} \cdot \frac{3}{\dots}$$

3

Sodium Side	$Nu=0.25+6.2 S_T - 0.007+0.032 S_T Pe^n$ $n=0.800-0.024 S_T, (S_T=P/d)$
Water Side	
Subcooled	$Nu=0.023 Re^{0.8} Pr^{0.4}$
Nucleate Boilnig	$h_B=Sh_b+Fh_c$ <p>S= Suppression factor, F:Martinelli parameter</p>

Film Boiling Super heated	$h_b = 0.00122 \left[\frac{k_l^{.79} C p_l^{.45} \rho_l^{.49}}{\sigma^{.5} \mu_l^{.29} \rho_g^{.24}} \right]^{0.68} \Delta T_{sat}^{.24} \Delta P_{sat}^{.75}$ $h_c = 0.023 \left[\frac{k}{d_i} \right] (1-x)^{.8} Re^{.85} Pr^{.4} di^{.1}$ $Nu = 0.0193 Re^{.8} Pr^{1.23} \left[x + (1-x) \frac{\rho_g}{\rho_f} \right]^{0.68} \left[\frac{\rho_g}{\rho_f} \right]^{0.068}$ $Nu = 0.0073 Re^{0.936} Pr^{0.61} di^{0.1}$
----------------------------------	---

3.3.

/ (He)

(gap)

gap (Gr>8000)

Gr 1~3%

, R_{CD}, L, r_{ii}, r_{io}

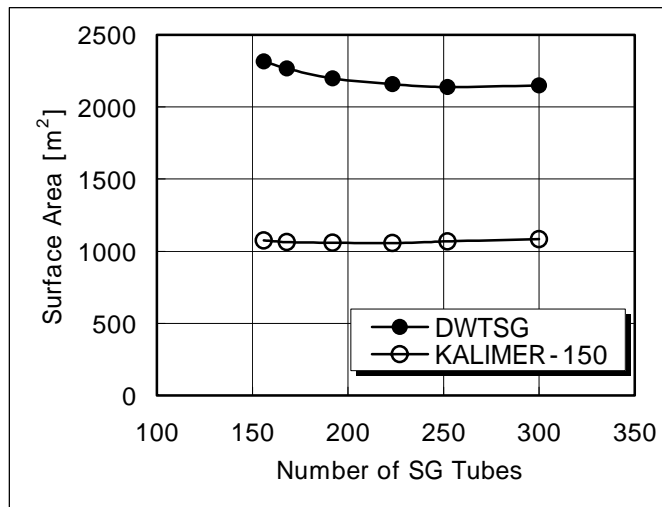
, r_{oi}, r_{oo}, k_w, k_g He

CFR He

15% (CFR=1.15).

$$R_{CD} = \left\{ \frac{\ln\left(\frac{r_{ii}}{r_{io}}\right)}{2\pi k_w L} + \frac{\ln\left(\frac{r_{oi}}{r_{io}}\right)}{2\pi k_g L} + \frac{\ln\left(\frac{r_{oo}}{r_{oi}}\right)}{2\pi k_w L} \right\} \times CFR$$

4. (K-150)
 KALIMER-150 SG 2 IHTS DWTSG
 2 . KALIMER-150
 DWTSG 2 가 , 가가



2 (IHTS)

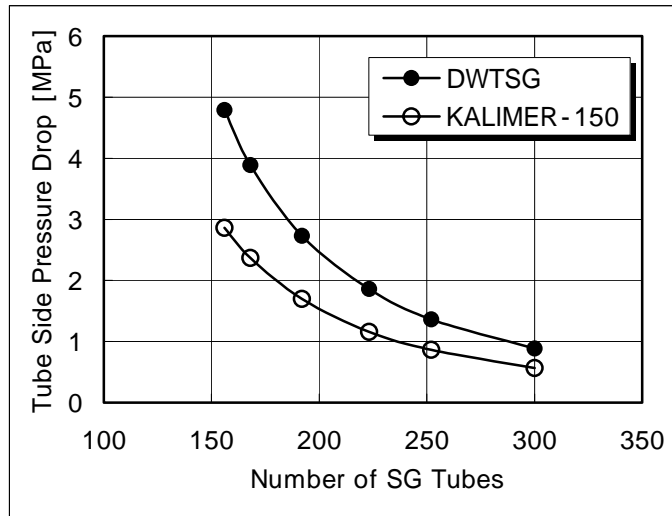
(가) 3.
 KALIMER 224 1.9MPa KALIMER (1.3
 MPa) 50% . DWTSG
 2 SG 가 .
 PHTS SG 4 .
 KALIMER 가
 , SG 가 . He
 DWTSG KALIMER- SG 95%
 SG
 55% 가 . DWTSG 가 KALIMER-150 (2) SG
 8 , DWTSG

70%

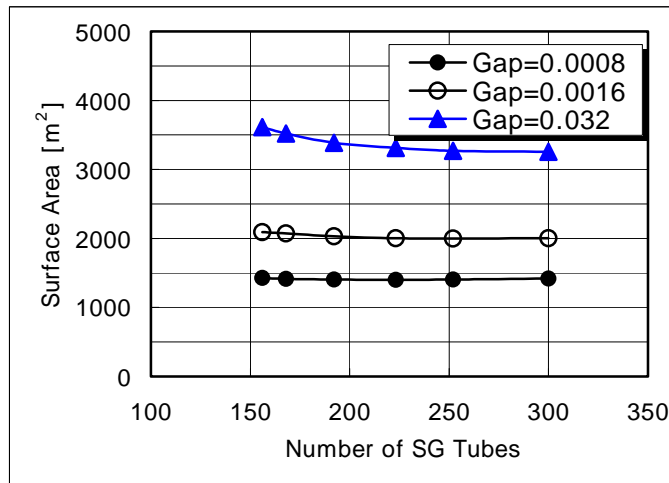
DWTSG

50%

DWTSG

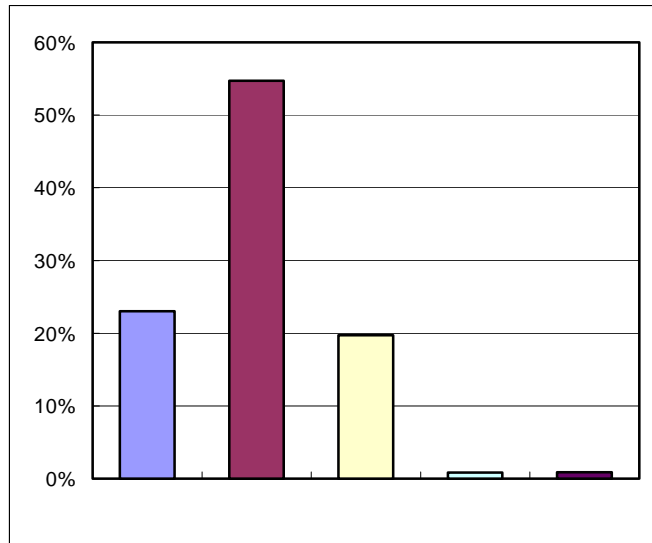


3



4

(PHTS)



5 DWTSG

5.

DWTSG

가

DWTSG

2

가

가

SG

가

SG

1. Gromov, et al, "Use of Lead-Bismuth Coolant in Nuclear Reactors and Accelerator-driven System", Nuclear Engineering Design, 173, 1997

2. Seong-O Kim "Evaluation of New Design Concepts for Steam Generators in

Sodium Cooled Liquid Metal Reactors”, Journal of the Korea Nuclear Society, Vo1. 35, No.2, April, 2003.

- 3 , , , “Lead-Bismuth ”, 2003 , 2003.
- 4 P.J Langford “ Alternative Concept Evaluation LMFBR Duplex - Tube Steam Generators”,WNET - 110,1974.
- 5 J.I. Sackett, “Operating and Test Experience with EBR-II, the IFR Prototype”, Progress in Nuclear Energy, Vol.3, No.1/2, 1997
- 6 Foster Wheeler Co “Preliminary Design: Duplex Tube Low Pressure Saturated Steam Generator for Large LMFBR Plant”, NP- 1219, Oct. 1979
- 7 Roy, P. and Dodson, G.R., Effects of Particulate Deposition on the Heat Transfer Coefficient of LMFBR Heat Exchanger, GE Report GEFT - 00265(L), 1977
8. KAERI, “KALIMER Conceptual Design Report”, KAERI/TR - 2204/2002, 2002