

2003

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Evaluations on Shutdown Cooling Performance Test for Korean Standard Nuclear Power Plants

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150

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가

KDSCNT1¹⁾

5

A B

가

가

가

가

ABSTRACT

The new acceptance criteria for shutdown cooling performance test for Korean Standard Nuclear Power Plant (KSNP) were developed using **KDSCNT1** code and to adequately evaluate the actual pre-core cooldown test conditions. The UCN 5 cooldown tests from hot shutdown to refueling modes under the pre-core hot functional test (HFT) conditions were performed using Shutdown Cooling System (SCS) Train A and B, respectively. The test results were evaluated as compared with the new acceptance criteria. These evaluations show that the new acceptance criteria appear to be reasonable to verify the SCS performance during the pre-core HFT cooldown stage, although there is a little deviation at the later stage of cooldown between the test result and the prediction. This deviation seems to be caused by the energy addition of the reactor coolant pump (RCP) operations and the heat removal of steam generators, which is not assumed in the prediction assumptions.

1.

(SCS)
36

U.S. NRC BTP RSB 5-1²⁾

350

200

24

350

125

() 가

가

가
가

40~25
가

가

가

75 /hr

가

5

가
A B

가

2.

가

가

2.1

NRC BTP RSB 5-1²⁾

3.5

가

32.5

가 350

200

2,3)

5,6

가

NRC BTP RSB 5-1

가

3.5

110 , 5.5

105 , 20.5

100 , 27.5

95

70 ~90

가

5,6

40

1,000~1,050gpm,
2,000~2,200gpm

40

1,150~1,200gpm ,

KDSCNT1

65 , 75 , 85 , 95 110

1

(2

110 , 17

105 ,

24

100)

8,000gpm

0~4,800gpm

20%

4.3

350

200

2.2

()
 3.5 가
 24 가 350 125
 5,6
 40 1,000~1,050gpm, 40
 1,150~1,200gpm , 2,000~2,200gpm
 . KDSCNT1
 가 65 , 75 , 85 , 95 110
 2
 (2 110 , 17 105 , 24
 100) ,
 8,000gpm , 0~4,800gpm
 22
 350 125
 . 3 5,6

3.

5 A B
 1, 2 3~10
 57 01A 65 B
 가 4 28 125 02A
 248 가 14
 5 가 A 248

4.

KDSCNT1 () ()

가
 A
 B
 2
 A B A
 가

가

A

가 가

B

B

(QHX)

4)

$$QHX = U * F * LMTD * AREA \dots\dots\dots(1)$$

U : (Btu/Hr- -ft²)
 AREA : (ft²)
 F :

$$F = \frac{\sqrt{R^2 + 1}}{R - 1} \ln\left(\frac{1 - P}{1 - PR}\right) / \ln\left(\frac{\frac{2}{P} - 1 - R + \sqrt{R^2 + 1}}{\frac{2}{P} - 1 - R - \sqrt{R^2 + 1}}\right) \dots\dots\dots(2)$$

P:
 R:
 LMTD :

$$LMTD = \frac{(TEMP - TCCW2) - (THX2 - TCCW1)}{\ln\left(\frac{TEMP - TCCW2}{THX2 - TCCW1}\right)} \dots\dots\dots(3)$$

TEMP :
 TCCW2 :
 THX2 :
 TCCW1 :

(U)

$$A = \frac{1}{U_{FINAL}} - C \dots\dots\dots(4)$$

$$TRRF = A \frac{TRRF}{TRRF + SRRF} \dots\dots\dots(5)$$

$$SRRF = A - TRRF \dots\dots\dots(6)$$

$$U = \frac{1}{TRRF \left(\frac{UFLOW}{FLOHXX}\right)^X + SRRF \left(\frac{SFLOW}{FLOCCW}\right)^Y + C} \dots\dots\dots(7)$$

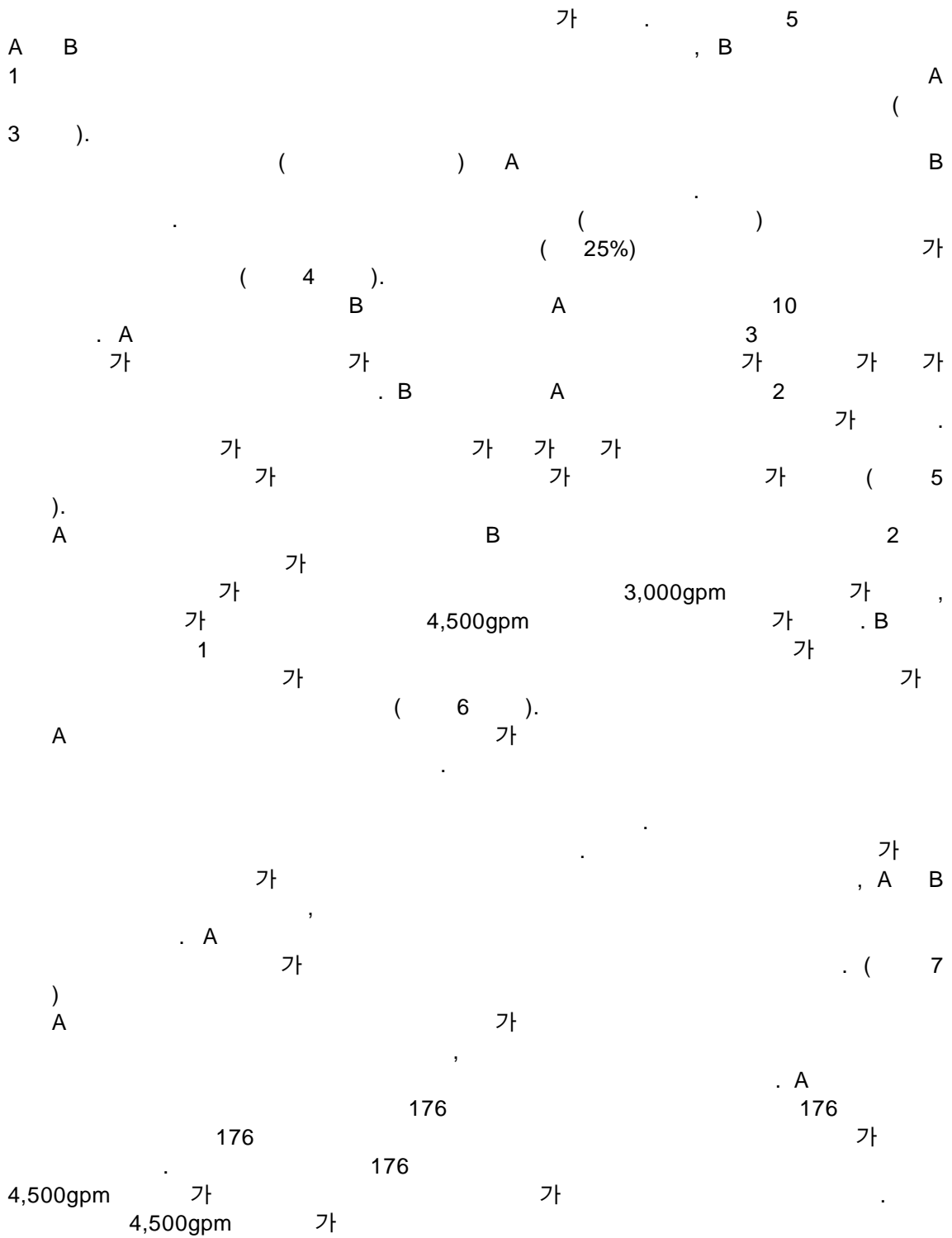
$$TFF = \frac{FLOHXX}{UFLOW} \dots\dots\dots(8)$$

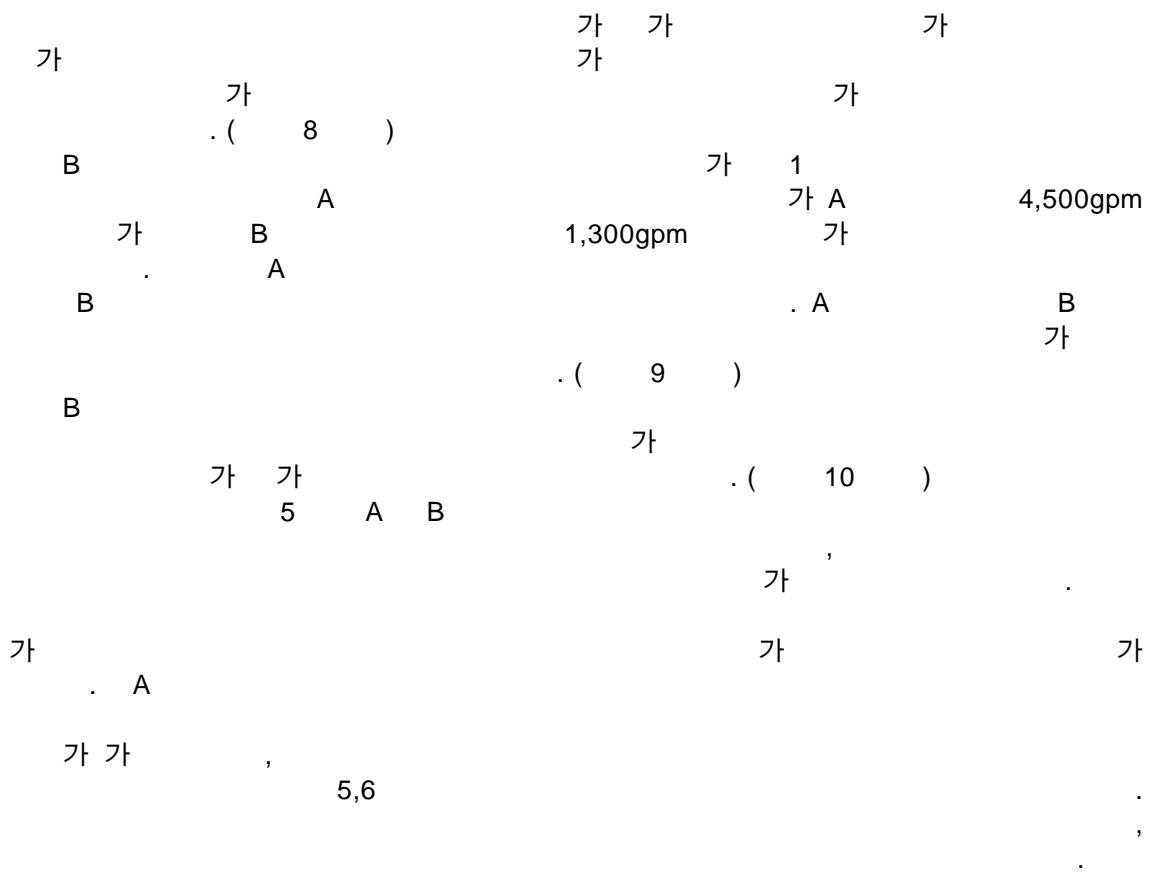
$$SFF = \frac{FLOCCW}{SFLOW} \dots\dots\dots(9)$$

U_{FINAL} :
 C :
 U_{FLOW} :

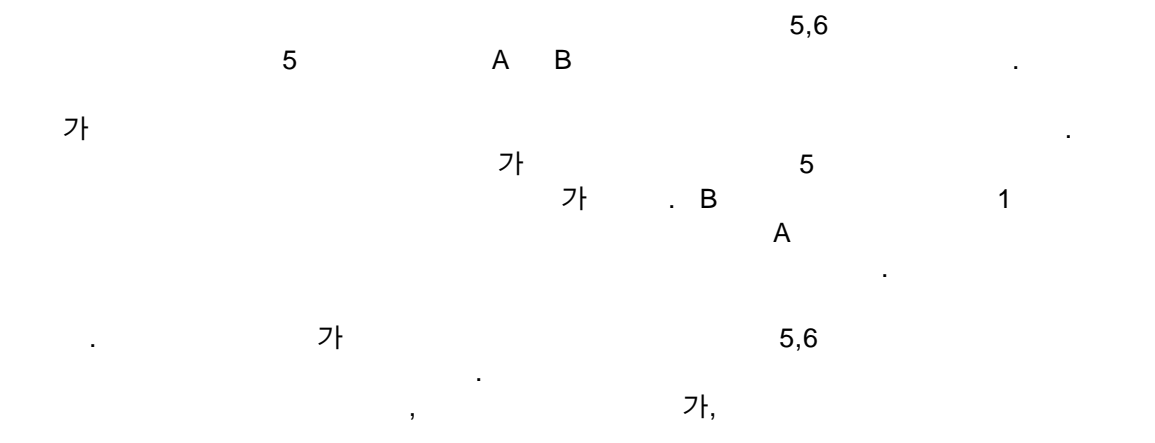
SFLOW :
FLOCCW :
FLOHXX :
X=0.7, Y=0.6

5.





6.



1. "Computer Code, KDSCNT1," Rev. 00, (1994).
2. "Design Requirements of the Residual Heat Removal System," RSB 5-1, NRC Branch Technical Position, USA, (1981).
3. "Design Criteria for Safe Shutdown Following Selected Design Basis Events in Light

Water Reactors,” ANSI/ANS 58.11 -1995, ANS, USA, (1995).

4. Chisholm, D., “Developments in Heat Exchanger Technology,” pp14-39, Applied Science Publishers (1980).

1.

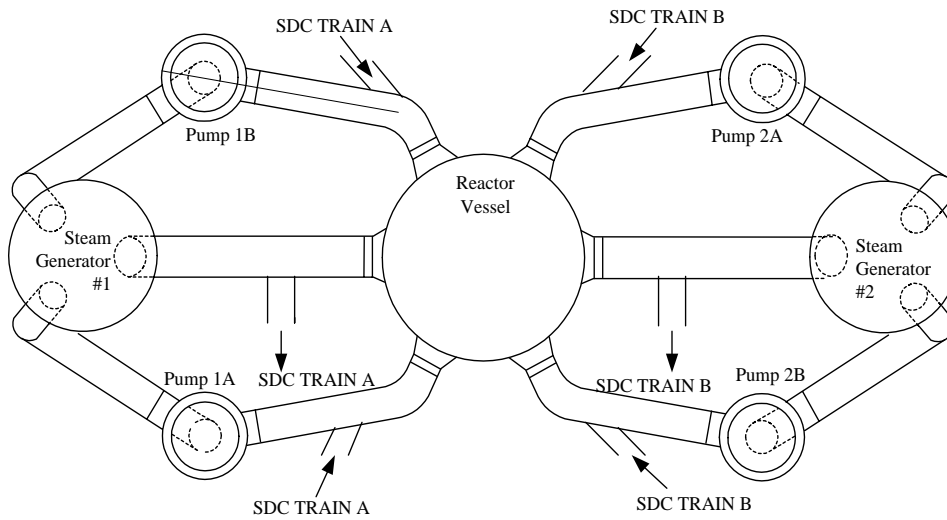
	350	200
		+
65	2.7	3.3
75	2.9	3.5
85	3.1	3.7
95	3.2	3.9
110	3.6	4.3

2.

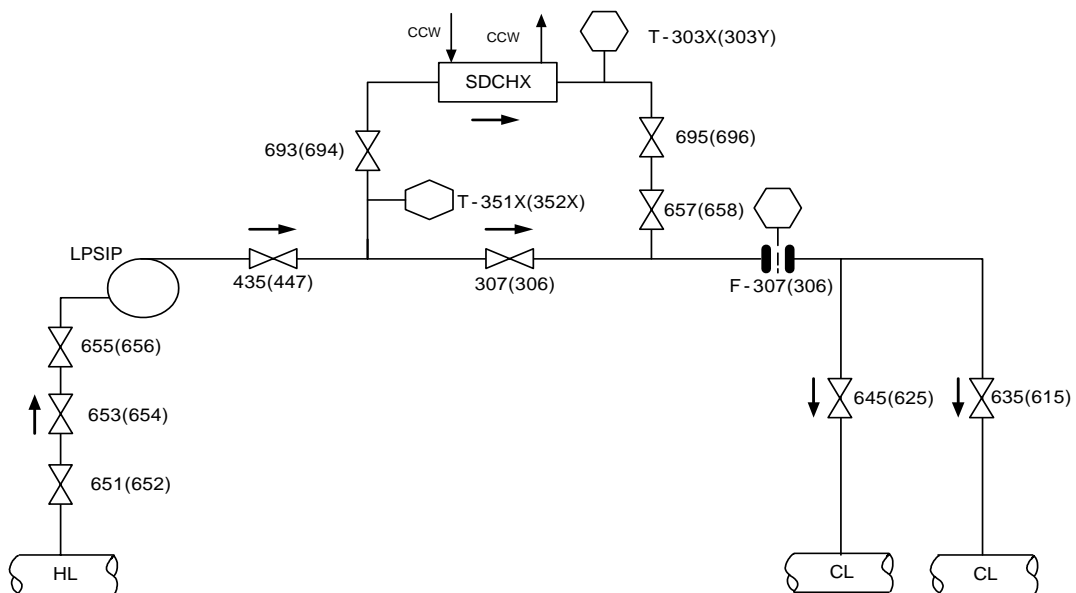
	350	125
		+
65	5.7	6.8
75	6.2	7.5
85	6.9	8.3
95	7.9	9.5
110	10.5	12.6

3.

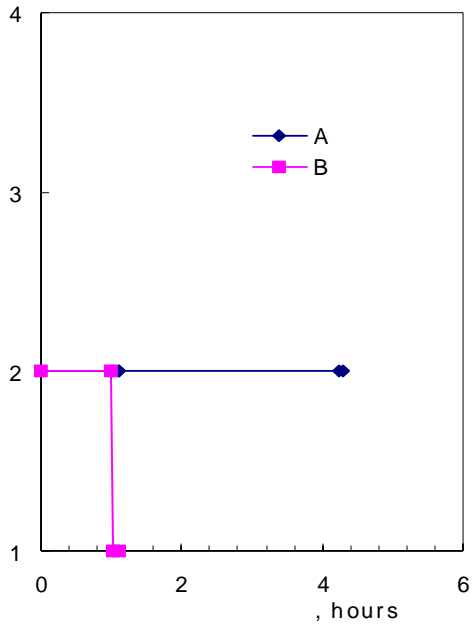
	5,6	5,6
	200	40 1,000 ~1,050gpm, 40 1,150~1,200gpm 2,000~2,200gpm 1 가 350 200
	24 가 350 125	40 1,000 ~1,050gpm, 40 1,150~1,200gpm 2,000~2,200gpm 2 가 350 200



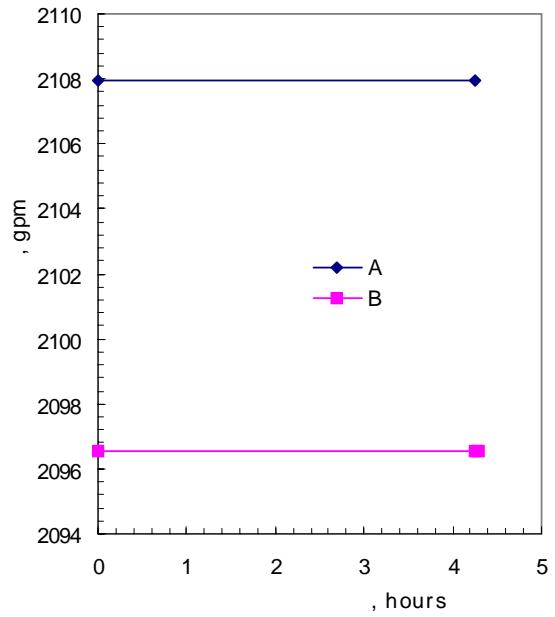
1.



2. 5

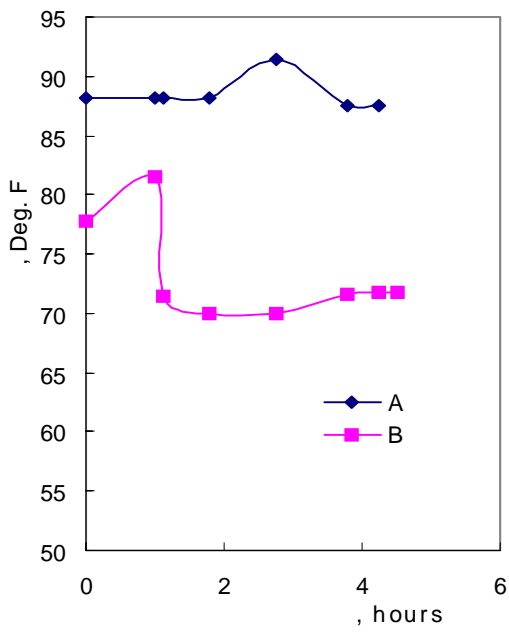


3.



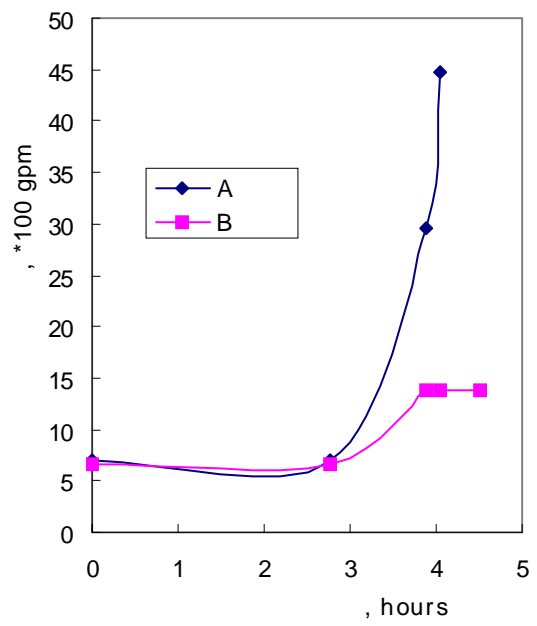
4.

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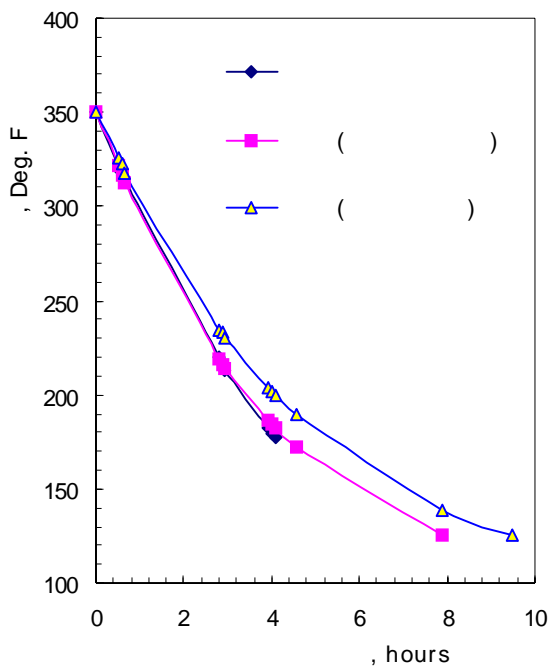


5.

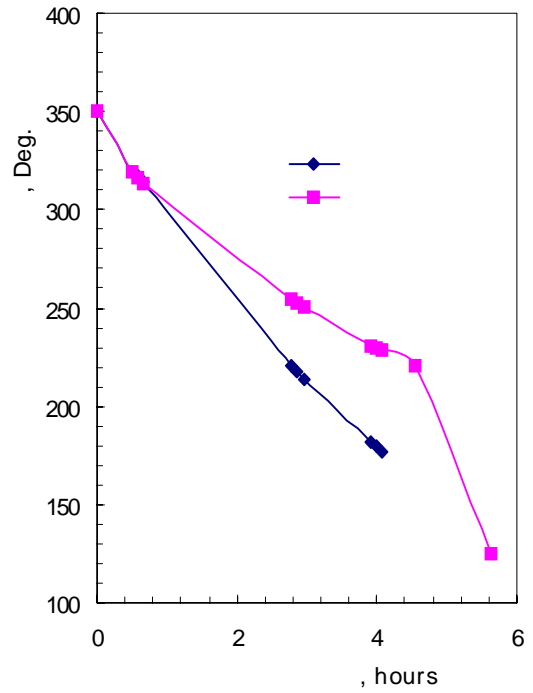
()



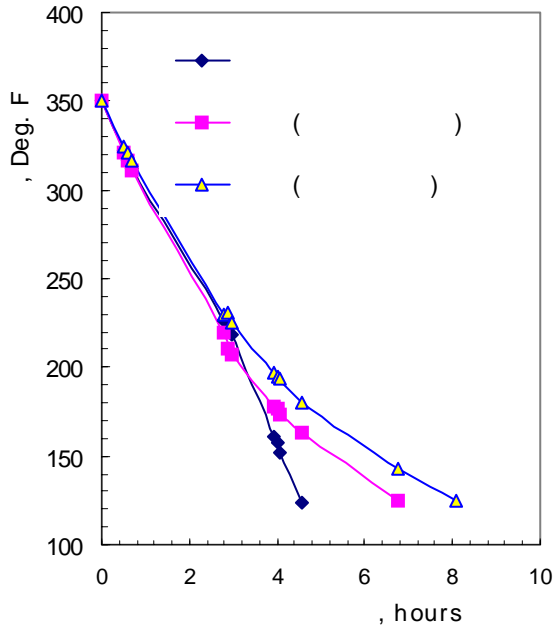
6.



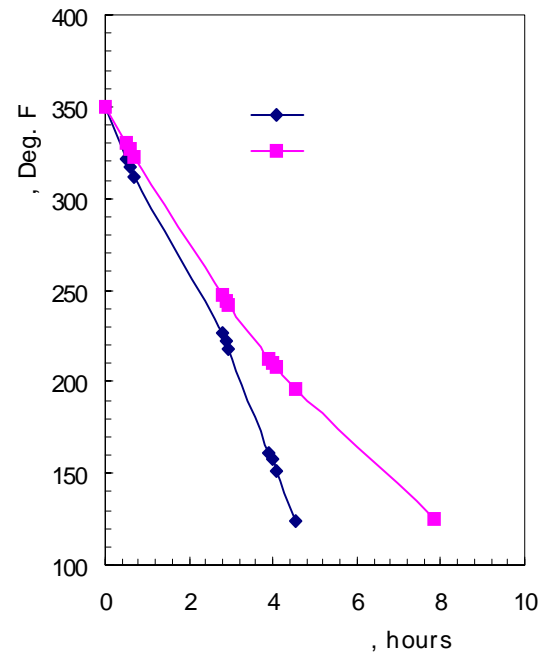
7. (A)



8. (A)



9. (B)



10. (B)