

Effect of Spring Properties on the Wear Behaviors of Nuclear Fuel Rod in High Temperature Distilled Water

150

가 가
가
third body abrasion
가
Load-bearing layer
가

Abstract

Sliding wear tests have been carried out in room and high temperature distilled water in order to evaluate the effect of test temperature on the wear behavior of nuclear fuel rods. In high temperature water, wear volume was dramatically decreased compared with room temperature water condition. However, test temperature did not have a significant effect on the the maximum wear depth of fuel rod. This means third body abrasion due to wear particles accelerates the increase of wear volume in room temperature water. But it is not enough to explain the difference of wear volume only with wear particle behavior between contact areas. To evaluate the cause of wear volume difference, a load-displacement test in high temperature water was conducted and we concluded that relatively low wear volume was affected by smaller slip displacement due to the decrease of spring stiffness.

1.

가 . 가 , 가
가 / 가 .
가 가
가 / 가
가 가
가 가
가 AECL , 90°
/ 가 가 /
 ,
가

2.

(1)

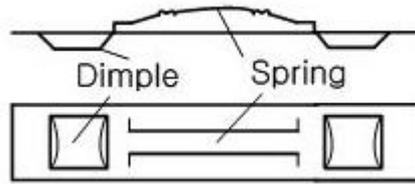
Zircaloy-4 Zirlo . 50 mm

1 . 가 가 가

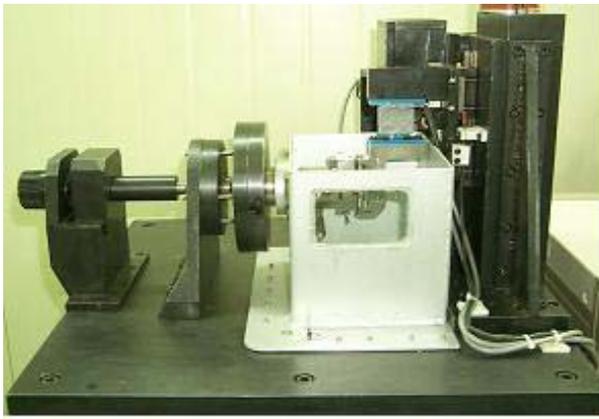
(2)

2 . 가
10 N 30 ~ 100 μ m , 30 Hz ,
10 (300)
2 가

가



1.



(a)



(b)

2.

(3)

2

[1, 2].

(4)

300

50

0.5mm

가

()

3.

(1)

2

3

가

가

가

가

가

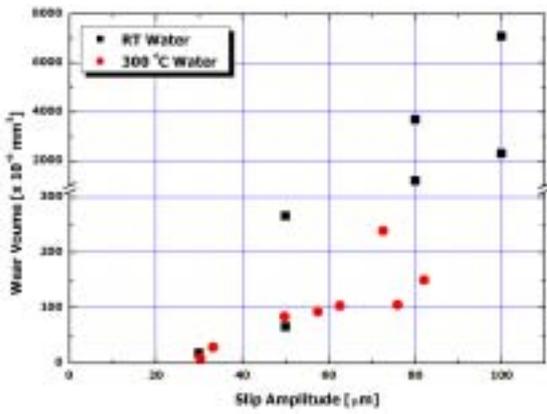
가 가

가 가 200

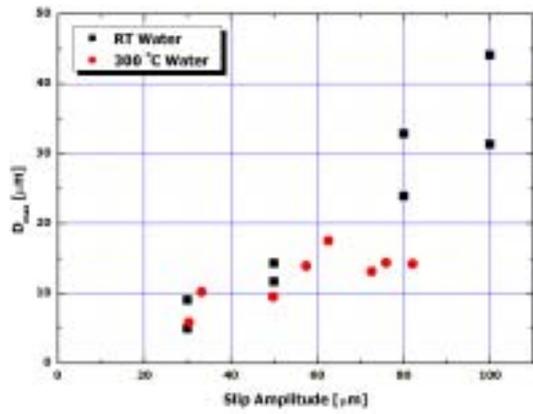
80 μm

가

가 [3].



3.



4.

(2)

4

가

third body abrasion

가

가

(3)

5

가

가

third body abrasion

가

가

가

가

third body abrasion

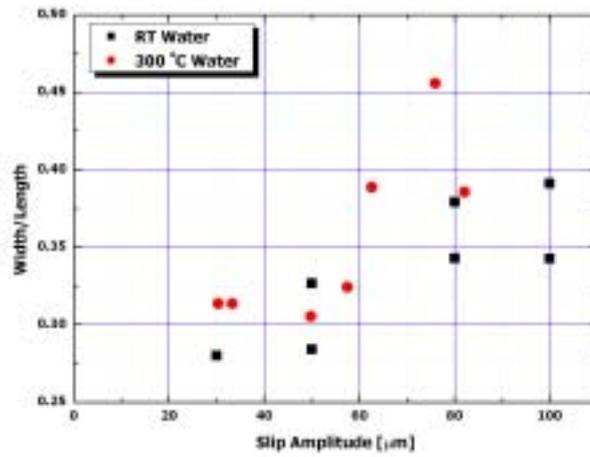


(a)



(b)

5.



6.

가

가

가

4.

(1)

가

가

가

가

가

()

(PLUS-7)

KAFD

(

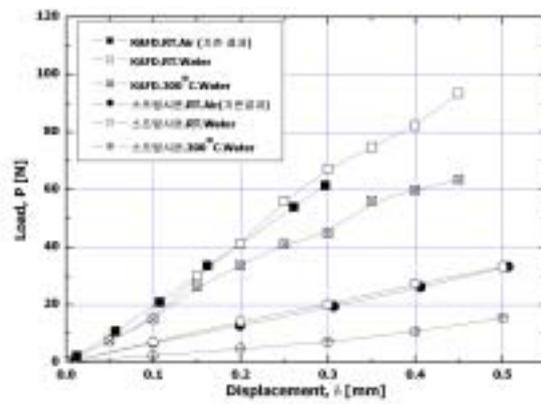
7)

가

가

가

가



7.

300

(2)

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- [3] F.M. Guerout 3, “Effect of temperature on steam generator fretting wear”, ASME PVP, Vol. 32 Flow-Induced Vibration (1996) 233-246.
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