

The Introduction of the verification technology and method for UF6 Cylinders

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1. Introduction

This paper provides the NDA technology and method to verify the contents of the UF6 cylinder at the nuclear fuel fabrication plant. The method is to meet the Agency's safeguards goals. The KNFC (Korea Nuclear Fuel Co. Ltd.) imports UF6 cylinders that are enriched at the foreign enrichment plants, according to requirements by KHNP (Korea Hydro & Nuclear Power Co. Ltd.). Then they are stored in the UF6 storage before transferring in the vaporization process. Therefore, the verification of uranium enrichment of UF6 cylinders is very important issue for the International Atomic Energy Agency (IAEA) and Korea Institute of Nuclear Nonproliferation and Control (KINAC) to verify all foreign receipts of low enriched uranium (LEU) needed for all LWRs. This paper introduces the technology and method necessary for verification of UF6 Cylinders for safeguards purposes and verification results for national safeguards inspection.

2. The Measurement of Uranium Enrichment

The important information for the quantitative nondestructive assay (NDA) of nuclear material is the full-energy peak measured from a sample because uranium and plutonium samples have a wide variety of isotopic compositions. This measurement is used to establish the fraction of fissile ^{235}U , commonly referred to as the uranium enrichment.

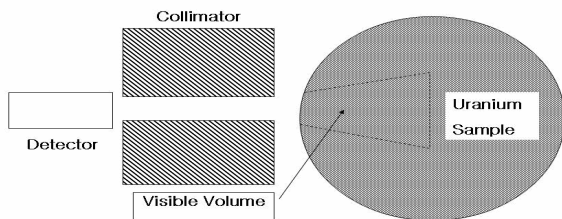


Fig 1. The basic elements of a gamma-ray uranium enrichment measurement setup

The primary radiation used in passive NDA of uranium samples is gamma radiation, which is usually dominated by emission from ^{235}U decay. The 186-keV gamma ray is the most frequently used signature to measure ^{235}U enrichment because it is the most prominent single gamma ray from any uranium sample enriched above natural uranium levels. The basic measurement procedure involves viewing a uranium sample through a collimated channel with a gamma-ray detector (Figure 1).

Table 1. Mean free paths and infinite thickness for 186-keV photons in uranium compounds.

| Uranium Compound | Density (g/cm ³) | Mean Free Path (cm) | "Infinite" Thickness (cm) |
|------------------|------------------------------|---------------------|---------------------------|
| Metal | 18.7 | 0.04 | 0.26 |
| UF6 (solid) | 4.7 | 0.20 | 1.43 |
| UO2 (sintered) | 10.9 | 0.07 | 0.49 |
| UO2 (powder) | 2.0 | 0.39 | 2.75 |
| U3O8 (powder) | 7.3 | 0.11 | 0.74 |
| Uranyl nitrate | 2.8 | 0.43 | 3.04 |

The "visible volume" of the sample is determined by the collimator, the detector geometry, and the mean free path of the 186-keV radiation in the sample material. Because of that, the mean free path and infinite-thickness values for the 186-keV gamma-ray should be reflected in commonly encountered uranium compounds (Table 1). The cylinder containing UF6 in solid forms vary in size and wall thickness. The large, high-density wall thickness of the cylinders means that minor variation in wall thickness can result in significant variation in gamma-ray count rate. Therefore, wall thickness of the cylinders must be measured by a thickness-gauge and reflected in measurement program to reduce the measurement error.

3. Verification of UF6 Cylinders in KNFC

3.1 Verification Criteria required by IAEA

IAEA started to perform SNRI (Short Notice Random Inspections) at KOR- from the beginning of 2007. An annual PIV and small number of SNRIs a year will be scheduled under Traditional Safeguards at the KOR- by Agency. All of nuclear material at KOR- is subject to the verification of PIV and 100% of nuclear transfer in or out the KOR- is subject to the verification of SNRIs.

All of UF6 cylinders subjected to the verification are verified with random medium detection probability for gross, partial and bias defects. And Inventory changes involving UF6 are reported to Agency through the mailbox declaration.

3.2 Measurement

UF6 cylinders are needed to be verified for partial defect. The partial defect means a fractional difference between the declared amount of nuclear material and the real amount of the material actually present.

Prior to the actual verification of UF6 cylinders, thickness measurements were performed on each cylinder wall. The thickness was utilized by the software in correcting for the attenuation of the 186-keV signal.

The actual verification of UF6 is divided into two parts. One is the verification of enrichment (Method F), the other is the verification of weight (Method B). The procedure of the verification and evaluation for partial defect is as follows;

- Measurement of weight by load cell
- Verification of enrichment with a single count time of 300 sec
- Calculation of d ([Declared weight of ^{235}U – Measured weight of ^{235}U] / Declared weight of ^{235}U)
- Comparison between |d| and 3δ (if $|d| \leq 3\delta$, results are satisfied)

3.3 UF6 Enrichment Verification System

The measurement system used in this work is the IMCG verifier (Inspection Multi-Channel Analyzer with Germanium Detector) produced by Canberra Industries.

Table 2. Measurement System

| | |
|-----------------------------|--|
| Detector | Low Energy Germanium (LEGe) |
| MCA | Inspector 2000 portable multi-Channel Analyzer |
| Software | IMCA 2000 operated in the GENIE 2000 environment, communication with MCA via USB |
| Computer | TOSHIBA, Window XP |
| Ultrasonic Thickness Gauges | Krautkramer Branson Model DM4 E |

The highly automated system includes total computer control of the MCA, automatic data storage, and analysis through a preset region of interest containing the 186-keV peak.

3.4 Result

During the Physical Inventory Verification(PIV) at KNFC in 2006, 31 UF6 cylinders were measured to verify the ^{235}U enrichment by using the detector system described in Table 2 (Table 3)

Table 3. Summary of UF6 cylinders ^{235}U enrichment measurement results by IMCG in 2006-PIV

| Operator's declared value | | | Inspector's measured value | | | Evaluation | |
|--------------------------------------|-----------------------------------|---------------------|----------------------------|--------------------------------|--------------|----------------|--|
| Enrichment of ^{235}U (wt%) | Weight of ^{235}U (kg/g) | Weight (kg/g) Gross | Weight (kg/g) Net | ^{235}U Enrichment(%) | Error d (%) | d <3δ (Yes/No) | |
| 3.803 | 52.644 | 2678.000 | 2045.1 | 3.739 | 1.8 | Y | |
| 3.798 | 57.458 | 2869.5 | 2235.9 | 3.925 | 3.2 | Y | |
| 3.8 | 23.933 | 1545.8 | 930.8 | 3.805 | 0.1 | Y | |
| 3.812 | 54.446 | 2112.869 | 2112.9 | 3.771 | 1.1 | Y | |
| 4.5 | 67.655 | 2223.7 | 2222.7 | 4.501 | 0 | Y | |
| 4.5 | 67.533 | 2219.7 | 2220.6 | 4.525 | 0.6 | Y | |
| 3.796 | 54.614 | 2128.651 | 2129.651 | 3.641 | 4.2 | Y | |
| 3.796 | 54.648 | 2130.198 | 2128.698 | 3.653 | 4 | Y | |
| 3.791 | 54.591 | 2130.559 | 2130.559 | 3.79 | 0 | Y | |
| 3.799 | 54.706 | 2130.605 | 2129.605 | 3.702 | 2.7 | Y | |
| 3.828 | 55.105 | 2129.651 | 2128.151 | 3.822 | 0 | Y | |
| 4.503 | 67.294 | 2210.3 | 2209 | 4.474 | 0.7 | Y | |
| 4.503 | 66.158 | 2173 | 2172 | 4.548 | 0.9 | Y | |
| 4.503 | 67.78 | 2226.7 | 2226.3 | 4.462 | 0.9 | Y | |

The verification results are summarized in Table 3 and the results by MMCG in 2005-PIV are summarized in Table 4 to compare with the accuracy of the equipments (IMCG and MMCG). In results by IMCG, measurement errors were significantly reduced. Also these results satisfied the criteria of safeguards inspection.

Table 4. Summary of UF6 cylinders ^{235}U enrichment measurement results by MMCG in 2005-PIV

| Operator's declared value | | | Inspector's measured value | | | Evaluation | |
|--------------------------------------|-----------------------------------|---------------------|----------------------------|--------------------------------|--------------|----------------|--|
| Enrichment of ^{235}U (wt%) | Weight of ^{235}U (kg/g) | Weight (kg/g) Gross | Weight (kg/g) Net | ^{235}U Enrichment(%) | Error d (%) | d <3δ (Yes/No) | |
| 3.812 | 54.35 | 2109.19 | 2110.694 | 3.9443 | 3.5 | Y | |
| 3.812 | 54.38 | 2110.56 | 2112.6 | 3.7848 | 0.6 | Y | |
| 3.804 | 54.149 | 210.7 | 2108.79 | 3.7498 | 1.4 | Y | |
| 3.794 | 54.103 | 2110.147 | 2109.65 | 3.8303 | 0.9 | Y | |
| 3.794 | 54.044 | 2108.240 | 2109.24 | 3.833 | 1.1 | Y | |
| 4.501 | 64.393 | 2116.0 | 2115 | 4.438 | 1.5 | Y | |
| 4.499 | 64.61 | 2124.1 | 2123.1 | 4.7908 | 6 | Y | |
| 4.5 | 68.702 | 2258.971 | 2258.97 | 4.6748 | 3.7 | Y | |
| 4.495 | 67.085 | 2208.12 | 2207.62 | 4.5421 | 1 | Y | |
| 4.495 | 67.071 | 2207.67 | 2208.17 | 4.529 | 0.8 | Y | |
| 4.594 | 67.068 | 2207.715 | 2208.22 | 4.5224 | 0.7 | Y | |
| 4.494 | 67.097 | 2208.67 | 2208.17 | 4.8152 | 6.6 | Y | |
| 4.514 | 67.26 | 2204.0 | 2204.3 | 4.3917 | 2.8 | Y | |

4. Conclusions

The verification of UF6 contents is very important issue for the Safeguards inspection purpose at the nuclear fuel fabrication plant. This paper shows the technology and method necessary for verification of UF6 contents.

Korea Institute of Nuclear Nonproliferation and Control (KINAC) replaced MMCG by IMCG to verify enrichment of UF6 cylinders to improve the quality of verification. As a result, the accuracy of the verification result and efficiency of the verification are evaluated to be increased. It is also expected that IMCG system is able to be applied to other types of nuclear materials to improve the quality of verification.

REFERENCES

- [1] IAEA, Integrated Safeguards Approach for Depleted, Natural and Low Enriched Uranium Conversion and Fabrication Plants, 2003.
- [2] Z. Starivich, Integrated Safeguards Approach for DNLEU Conversion and Fabrication Plant in ROK, IAEA, 2006
- [3] R. Zarucki, Introduction to the Conceptual Framework for Implementation of IS in ROK. IAEA, 2006